Nuclear Operating Company
South Texas Project Electric Generating Station P.O. Box 289 Wadsworth, Texas 77483

February 21, 2000
NOC-AE-000745
File No.: G03.08
10CFR50

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

South Texas Project
Units 1 and 2
Docket Nos. STN 50-498, STN 50-499
Response to Request for Additional Information: Generic Letter 96-06

References: 1) Request for Additional Information, Thomas W. Alexion, Nuclear Regulatory Commission, to William T. Cottle, STP Nuclear Operating Company, dated December 20, 1999

2) Response to Request for Additional Information, S. E. Thomas, South Texas Project, to NRC Document Control Desk, dated November 11, 1997 (ST-HL-AE-5780)

Pursuant to the Nuclear Regulatory Commission request for additional information (reference 1), the South Texas Project submits the attached report, "Containment Air Cooler Heat Transfer During Loss of Coolant Accident with Loss of Offsite Power Conditions," November 1996, prepared by Numerical Applications, Inc. The report was referenced in the response to Generic Letter 96-06, "Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions" (reference 2).

Pursuant to a telephone conversation with Nuclear Regulatory Commission staff conducted June 24, 1999, the South Texas Project submits the following responses to questions regarding the status of calculations performed to determine the potential for piping overpressurization. The calculations were previously discussed in reference 2.

- *As stated in Attachment 1 of the November 11, 1997, submittal, additional calculations are expected to be performed to determine if a design basis accident will result in pressures less than the actual benchset pressure. What is the status of the calculations?*

  The calculations for the impact of a design basis accident on piping pressure have been completed and the resulting stresses are within the ASME Code allowable limits for a faulted condition based on Appendix I of ASME Section III, 1974 Edition. Consequently, there is no need to take credit for the air-operated valves for overpressure protection.

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The final pipe pressurization design calculation applies to both the current steam generators and the planned replacement steam generators.

- *As stated in Attachment 2 of the November 11, 1997, submittal, listed pressure values were generated using a preliminary version of the code developed to perform the pipe pressurization analyses. These results and the results of the benchmarking study were reviewed and found acceptable by the South Texas Project. The peak pressures were not expected to change; however, the code had not yet been verified, so changes were possible. What is the status of the code verification and the calculated pressure values?*

  A revised version of the code has been verified in accordance with station procedures. There were changes in peak pressures calculated using the verified code; however, the resulting stresses remain less than the ASME Code allowable limits for a faulted condition based on Appendix I of ASME Section III, 1974 Edition .

If there are any questions, please contact either Mr. P. L. Walker at (361) 972-8392 or me at (361) 972-7795.

D. G. Leazar

D. A. Leazar
Manager,
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PLW

Attachment: Containment Air Cooler Heat Transfer During Loss of Coolant Accident with Loss of Offsite Power Conditions, November 1996, prepared by Numerical Applications, Inc.

cc:

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