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Quad Cities Generating Station
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February 11, 2000

SVP-00-036

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555-0001

Quad Cities Nuclear Power Station, Units 1 and 2
Facility Operating License Nos. DPR-29 and DPR-30
NRC Docket Nos. 50-254 and 50-265

Subject: Monthly Operating Report

In accordance with Generic Letter 97-02 and Technical Specification 6.9.5, "Monthly Operating Reports," we are submitting the Monthly Operating Report for Quad Cities Nuclear Power Station, Units 1 and 2. This report covers the period of January 1, 2000 to January 31, 2000.

Should you have any questions concerning this letter, please contact Mr. C.C. Peterson at (309) 654-2241, extension 3609.

Respectfully,

A handwritten signature in black ink, appearing to read "Joel P. Dimmette, Jr.", written over a large, stylized flourish that extends to the right.

Joel P. Dimmette, Jr.
Site Vice President
Quad Cities Nuclear Power Station

Attachment

cc: Regional Administrator – NRC Region III
NRC Senior Resident Inspector – Quad Cities Nuclear Power Station

ATTACHMENT

QUAD CITIES NUCLEAR POWER STATION UNITS 1 AND 2
MONTHLY OPERATING REPORT

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

FACILITY OPERATING LICENSE NOS. DPR-29 AND DPR-30

NRC DOCKET NOS. 50-254 AND 50-265

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I. INTRODUCTION

Quad Cities Nuclear Power Station is composed of two Boiling Water Reactors and Steam Turbine/Generators, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and MidAmerican Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/ Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Lynne Hamilton and Debra Kelley, telephone number 309-654-2241, extensions 3114 and 2240, respectively.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One began the month of January operating at full power. Unit One operated throughout the month at full power with minor down power operations for normal maintenance and surveillance testing.

B. Unit Two

Quad Cities Unit Two began the month of January in coastdown at the end of Q2O15. On January 21, at 12:01 a.m., Unit Two Turbine Generator was separated from the grid in preparation for Q2R15. At 1:28 a.m., Unit 2 Reactor was made subcritical. For the remainder of the month, refueling activities were performed.

III. OPERATING DATA STATISTICS

A. Unit One Operating Data Report for January 2000

DOCKET NO.: 50-254
 DATE: February 11, 2000
 COMPLETED BY: Lynne Hamilton
 TELEPHONE: (309) 654-2241

OPERATING STATUS

0000 010100

1. REPORTING PERIOD: 2400 013100 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
 DESIGN ELECTRICAL RATING (MWe-NET): 789

UNIT ONE	THIS MONTH	YTD	CUMULATIVE
3. NUMBER OF HOURS THE REACTOR WAS CRITICAL	744.00	744.00	185786.30
4. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
5. HOURS GENERATOR ON-LINE	744.00	744.00	180549.60
6. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
7. GROSS THERMAL ENERGY GENERATED (MWH)	1864584.00	1864584.00	398048302.44
8. GROSS ELECTRICAL ENERGY GENERATED (MWH)	605957.00	605957.00	128809469.00
9. NET ELECTRICAL ENERGY GENERATED (MWH)	580043.00	580043.00	116215485.00
10. REACTOR SERVICE FACTOR	100.00	100.00	76.22
11. REACTOR AVAILABILITY FACTOR	100.00	100.00	77.63
12. UNIT SERVICE FACTOR	100.00	100.00	74.07
13. UNIT AVAILABILITY FACTOR	100.00	100.00	74.45
14. UNIT CAPACITY FACTOR (Using MDC)	101.38	101.38	62.00
15. UNIT CAPACITY FACTOR (Using Design MWe)	98.81	98.81	60.43
16. UNIT FORCED OUTAGE RATE	0.00	0.00	6.78

III. OPERATING DATA STATISTICS

B. Unit Two Operating Data Report for January 2000

DOCKET NO.: 50-265
 DATE: February 11, 2000
 COMPLETED BY: Lynne Hamilton
 TELEPHONE: (309) 654-2241

OPERATING STATUS

0000 010100

1. REPORTING PERIOD: 2400 013100 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX. DEPEND. CAPACITY: 769
 DESIGN ELECTRICAL RATING (MWe-NET): 789

UNIT TWO	THIS MONTH	YTD	CUMULATIVE
3. NUMBER OF HOURS THE REACTOR WAS CRITICAL	481.50	481.50	178709.90
4. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
5. HOURS GENERATOR ON-LINE	480.00	480.00	174084.05
6. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
7. GROSS THERMAL ENERGY GENERATED (MWH)	1067375.76	1067375.76	383164083.66
8. GROSS ELECTRICAL ENERGY GENERATED (MWH)	343489.00	343489.00	122951004.00
9. NET ELECTRICAL ENERGY GENERATED (MWH)	327605.00	327605.00	116691337.00
10. REACTOR SERVICE FACTOR	64.72	64.72	73.79
11. REACTOR AVAILABILITY FACTOR	64.72	64.72	75.03
12. UNIT SERVICE FACTOR	64.52	64.52	71.88
13. UNIT AVAILABILITY FACTOR	64.52	64.52	72.17
14. UNIT CAPACITY FACTOR (Using MDC)	57.26	57.26	62.66
15. UNIT CAPACITY FACTOR (Using Design MWe)	55.81	55.81	61.07
16. UNIT FORCED OUTAGE RATE	0.00	0.00	10.46

IV. UNIT SHUTDOWNS

A. Unit One Shutdowns for January 2000

DOCKET NO.: 50-254
DATE: February 11, 2000
COMPLETED BY: Lynne Hamilton
TELEPHONE: (309) 654-2241

No.	DATE	TYPE FOR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	CORRECTIVE ACTIONS/COMMENTS
						None for the month of January.

Legend:

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other (Explain)

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

IV. UNIT SHUTDOWNS

B. Unit Two Shutdowns for January 2000

DOCKET NO.: 50-265
DATE: February 11, 2000
COMPLETED BY: Lynne Hamilton
TELEPHONE: (309) 654-2241

No.	DATE	TYPE FOR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	CORRECTIVE ACTIONS/COMMENTS
2000-01	000121	S	264	C	1	Q2R15 Refuel Outage

Legend:

(1) Reason

- A – Equipment Failure (Explain)
- B – Maintenance or Test
- C – Refueling
- D – Regulatory Restriction
- E – Operator Training/License Examination
- F – Administrative
- G – Operational Error (Explain)
- H – Other (Explain)

(2) Method

- 1 – Manual
- 2 – Manual Trip/Scram
- 3 – Automatic Trip/Scram
- 4 – Continuation
- 5 – Other (Explain)

V. AMENDMENTS TO FACILITY LICENSE OR TECHNICAL SPECIFICATIONS

Technical Specification Amendment No. 193 was issued on January 28, 2000 to Facility Operating License No. DPR-29 and Amendment No. 189 to Facility Operating License No. DPR-30 for Quad Cities Nuclear Power Station, Units 1 and 2, respectively.

The amendments revise Technical Specification (TS) 2.2 "Limiting Safety System Settings," and TS 3/4.1.A. "Reactor Protection System," to remove an anticipatory reactor trip signal, the turbine Electro-Hydraulic Control (EHC) low oil pressure trip, from the Reactor Protection System (RPS) trip function requirements.

Technical Specification Amendment No. 194 was issued on January 28, 2000 to Facility Operating License No. DPR-29 and Amendment No. 190 to Facility Operating License No. DPR-30 for Quad Cities Nuclear Power Station, Units 1 and 2, respectively.

The change to TS Section 3/4.1.A, "Reactor Protection System" (RPS), modifies the Surveillance Requirements (SR) for RPS Functional Unit 3, "Reactor Vessel Steam Dome Pressure – High." This change supports the licensee's planned upgrade to the subject instrumentation from pressure switches (Barksdale) to analog trip units (Rosemount). This change also adds a 31-day trip unit calibration consistent with other installed analog trip unit devices.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on the requirements set forth in Technical Specification 6.9.A.5.

A. Main Steam Relief Valve Operations

There were no Relief Valve Operations during the reporting period.