



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 2
DOCKET NO. 50-311**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for January 2000 is attached.

Sincerely,

A handwritten signature in black ink, appearing to read "M. B. Bezilla for HBB".

M. B. Bezilla
Vice President - Operations

/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

DOCKET NO.: 50-311
 UNIT: Salem 2
 DATE: 2/15/00
 COMPLETED BY: R. Knieriem
 TELEPHONE: (856) 339-1782

Reporting Period: January 2000

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
Maximum Dependable Capacity (MWe-Net)
No. of hours reactor was critical
No. of hours generator was on line (service hours)
Unit reserve shutdown hours
Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
744	744	96849
744	744	93526
0.0	0.0	0.0
817411	817411	93739629

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 2 began the month of January 2000, operating at 80% power as a contingency in anticipation of Y2K related problems affecting the power grid. Unit 2 returned to full power on January 1, and operated at full power for the remainder of the month.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 2 GENERATING STATION

MONTH: January 2000

The following items completed during **January 2000** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

Modification 2EE-0330, Pkg. 3, Rev. 0, 23 Service Water Traveling Screen Replacement

This modification improved the existing Service Water Traveling Screens by replacing the screen material with a new Smooth-Tex design and the existing three nozzle type screen wash headers were replaced with new assemblies having six. The modification also added air-operated valves to isolate the safety-related portion of the Service Water system from the non-Safety-related screen wash portion of the system.

Review of this modification under 10CFR50.59 was required because the upgrade of the 23 Service Water Traveling Screen constituted a change to the facility as described in the SAR. This upgrade improved the operation of the traveling screens and enhanced their ability to carry out their design function. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new

failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Temporary Modifications - Summary of Safety Evaluations

There were no changes in this category implemented during January 2000.

Procedures - Summary of Safety Evaluations

Procedure SC.MD-CM.CBV-0002(Q), Revision 2, Rod Drive Ventilation Fan Maintenance

This change revised the Rod Drive Ventilation Fan Maintenance procedure to address the replacement of a Rod Drive Ventilation Fan at power.

Review of this procedure change under 10CFR50.59 was required because the revision of the Rod Drive Ventilation Fan Maintenance procedure, to permit the replacement of a Rod Drive Ventilation Fan at power, constituted a change to the facility as described in the SAR. This change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

UFSAR Change Notices - Summary of Safety Evaluations

There were no changes in this category implemented during January 2000.

Deficiency Reports - Summary of Safety Evaluations

There were no changes in this category implemented during January 2000.

Other - Summary of Safety Evaluations

There were no changes in this category implemented during January 2000.