

February 10, 2000

Template # NRR-058

Mr. James Scarola, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

SUBJECT: SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1 - ISSUANCE OF
AMENDMENT RE: CORE OPERATING LIMITS REPORT METHODOLOGY
REFERENCES (TAC NO. MA6252)

Dear Mr. Scarola:

The Nuclear Regulatory Commission has issued Amendment No. 94 to Facility Operating License No. NPF-63 for the Shearon Harris Nuclear Power Plant (HNP), Unit No. 1, in response to your request dated August 4, 1999, as supplemented on December 3, 1999, and January 11, 2000. This amendment revises Technical Specification 6.9.1.6.2 to incorporate analytical methodology references which are used to determine core operating limits.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

/RA/

Richard J. Laufer, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

1. Amendment No. 94 to NPF-63
2. Safety Evaluation

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 10, 2000

Mr. James Scarola, Vice President
Shearon Harris Nuclear Power Plant
Carolina Power & Light Company
Post Office Box 165, Mail Code: Zone 1
New Hill, North Carolina 27562-0165

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A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's regular bi-weekly Federal Register notice.

Sincerely,

A handwritten signature in cursive script that reads "Richard J. Laufer".

Richard J. Laufer, Project Manager, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-400

Enclosures:

1. Amendment No. 94 to NPF-63
2. Safety Evaluation

cc w/enclosures:

See next page



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CAROLINA POWER & LIGHT COMPANY, et al.

DOCKET NO. 50-400

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 94
License No. NPF-63

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Carolina Power & Light Company, (the licensee), dated August 4, 1999, as supplemented on December 3, 1999, and January 11, 2000, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Facility Operating License No. NPF-63 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, as revised through Amendment No. 94, are hereby incorporated into this license. Carolina Power & Light Company shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 60 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Richard P. Correia, Chief, Section 2
Project Directorate II
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: February 10, 2000

ATTACHMENT TO LICENSE AMENDMENT NO. 94

FACILITY OPERATING LICENSE NO. NPF-63

DOCKET NO. 50-400

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages

Insert Pages

6-24

6-24

6-24a

6-24a

6-24b

6-24b

6-24c

6-24c

ADMINISTRATIVE CONTROLS

6.9.1.6 CORE OPERATING LIMITS REPORT

6.9.1.6.1 Core operating limits shall be established and documented in the CORE OPERATING LIMITS REPORT (COLR), plant procedure PLP-106, prior to each reload cycle, or prior to any remaining portion of a reload cycle, for the following:

- a. SHUTDOWN MARGIN limits for Specification 3/4.1.1.2.
- b. Moderator Temperature Coefficient Positive and Negative Limits and 300 ppm surveillance limit for Specification 3/4.1.1.3.
- c. Shutdown Bank Insertion Limits for Specification 3/4.1.3.5.
- d. Control Bank Insertion Limits for Specification 3/4.1.3.6.
- e. Axial Flux Difference Limits for Specification 3/4.2.1.
- f. Heat Flux Hot Channel Factor, $F_{\Delta H}^{RTP}$, $K(Z)$, and $V(Z)$ for Specification 3/4.2.2.
- g. Enthalpy Rise Hot Channel Factor, $F_{\Delta H}^{RTP}$, and Power Factor Multiplier, $PF_{\Delta H}$ for Specification 3/4.2.3.
- h. Boron Concentration for Specification 3/4.9.1.

6.9.1.6.2 The analytical methods used to determine the core operating limits shall be those previously reviewed and approved by the NRC at the time the reload analyses are performed, and the approved revision number shall be identified in the COLR.

- a. XN-75-27(P)(A), "Exxon Nuclear Neutronics Design Methods for Pressurized Water Reactors," approved version as specified in the COLR.

(Methodology for Specification 3.1.1.2 - SHUTDOWN MARGIN - MODES 3, 4 and 5, 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor, and 3.9.1 - Boron Concentration).
- b. ANF-89-151(P)(A), "ANF-RELAP Methodology for Pressurized Water Reactors: Analysis of Non-LOCA Chapter 15 Events," approved version as specified in the COLR.

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- c. XN-NF-82-21(P)(A), "Application of Exxon Nuclear Company PWR Thermal Margin Methodology to Mixed Core Configurations," approved version as specified in the COLR.

(Methodology for Specification 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).

ADMINISTRATIVE CONTROLS

6.9.1.6 CORE OPERATING LIMITS REPORT (Continued)

- d. XN-75-32(P)(A). "Computational Procedure for Evaluating Fuel Rod Bowing," approved version as specified in the COLR.
(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- e. Steam Line Break Methodology as defined by:
EMF-84-093(P)(A). "Steam Line Break Methodology for PWRs," approved version as specified in the COLR.
ANF-84-093(P)(A). "Steam Line Break Methodology for PWRs," approved version as specified in the COLR.
(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- f. SEM/PWR Large Break LOCA Evaluation Model as defined by:
EMF-2087(P)(A). "SEM/PWR-98: ECCS Evaluation Model for PWR LBLOCA Applications," approved version as specified in the COLR.
XN-NF-81-58(P)(A). "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model," approved version as specified in the COLR.
ANF-81-58(P)(A). "RODEX2 Fuel Rod Thermal Mechanical Response Evaluation Model," approved version as specified in the COLR.
XN-NF-82-06(P)(A). "Qualification of Exxon Nuclear Fuel for Extended Burnup," approved version as specified in the COLR.
ANF-88-133(P)(A). "Qualification of Advanced Nuclear Fuels' PWR Design Methodology for Rod Burnups of 62 Gwd/MTU," approved version as specified in the COLR.
XN-NF-85-92(P)(A). "Exxon Nuclear Uranium Dioxide/Gadolinia Irradiation Examination and Thermal Conductivity Results," approved version as specified in the COLR.
EMF-92-116(P)(A). "Generic Mechanical Design Criteria for PWR Fuel Designs," approved version as specified in the COLR.
(Methodology for Specification 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- g. XN-NF-78-44(P)(A). "A Generic Analysis of the Control Rod Ejection Transient for Pressurized Water Reactors," approved version as specified in the COLR.
(Methodology for Specification 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, and 3.2.2 - Heat Flux Hot Channel Factor).

ADMINISTRATIVE CONTROLS

6.9.1.6 CORE OPERATING LIMITS REPORT (Continued)

- h. ANF-88-054(P)(A), "PDC-3: Advanced Nuclear Fuels Corporation Power Distribution Control for Pressurized Water Reactors and Application of PDC-3 to H. B. Robinson Unit 2," approved version as specified in the COLR.

(Methodology for Specification 3.2.1 - Axial Flux Difference, and 3.2.2 - Heat Flux Hot Channel Factor).
- i. WCAP-9272-P-A, "WESTINGHOUSE RELOAD SAFETY EVALUATION METHODOLOGY", July 1985 (W Proprietary).

(Methodology for Specification 3.1.1.2 - SHUTDOWN MARGIN - MODES 3, 4 AND 5, 3.2.2 - Heat Flux Hot Channel Factor and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- j. WCAP-10266-P-A, Rev. 2, "The 1981 Version of the WESTINGHOUSE ECCS EVALUATION MODEL USING THE BASH CODE", March 1987 (W Proprietary).

(Methodology for Specification 3.2.2 - Heat Flux Hot Channel Factor).
- k. WCAP-11837-P-A, "EXTENSION OF METHODOLOGY FOR CALCULATING TRANSITION CORE DNBR PENALTIES", January 1990 (W Proprietary).

(Methodology for Specification 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- l. EMF-92-081(P)(A), "Statistical Setpoint/Transient Methodology for Westinghouse Type Reactors," approved version as specified in the COLR.

(Methodology for Specification 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- m. EMF-92-153(P)(A), "HTP: Departure from Nucleate Boiling Correlation for High Thermal Performance Fuel," approved version as specified in the COLR.

(Methodology for Specification 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).
- n. XN-NF-82-49(P)(A), "Exxon Nuclear Company Evaluation Model EXEM PWR Small Break Model," approved version as specified in the COLR.

(Methodology for Specification 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).

ADMINISTRATIVE CONTROLS

6.9.1.6 CORE OPERATING LIMITS REPORT (Continued)

- o. EMF-96-029(P)(A), "Reactor Analysis Systems for PWRs," approved version as specified in the COLR.

(Methodology for Specification 3.1.1.2 - SHUTDOWN MARGIN - MODES 3, 4 and 5, 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor, and 3.9.1 - Boron Concentration).

6.9.1.6.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements, shall be provided, upon issuance for each reload cycle, to the NRC Document Control Desk; with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the NRC in accordance with 10CFR50.4 within the time period specified for each report.

6.10 DELETED

(PAGE 6-25 DELETED)



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

CAROLINA POWER & LIGHT COMPANY

SHEARON HARRIS NUCLEAR POWER PLANT, UNIT 1

DOCKET NO. 50-400

1.0 INTRODUCTION

By letter dated August 4, 1999, as supplemented on December 3, 1999, and January 11, 2000, Carolina Power & Light Company (CP&L, the licensee) requested a revision to the Technical Specifications (TS) for the Shearon Harris Nuclear Power Plant (HNP). The proposed amendment would revise TS 6.9.1.6.2 to incorporate analytical methodology references which are used to determine core operating limits. The analytical methodologies to be referenced are documented in topical reports which have been accepted by the Nuclear Regulatory Commission for referencing in licensing applications. In addition to revising some of the references, the licensee proposed changing the format for other cited references in TS 6.9.1.6.2 for consistency.

2.0 BACKGROUND

HNP TS Section 6.9.1.6 provides information regarding the Core Operating Limits Report (COLR). HNP plant procedure PLP-106, "Technical Specification Equipment List Program and Core Operating Limits Report," implements the cycle-specific core parameter limits. These limits are developed using the NRC-approved methodologies specified in TS 6.9.1.6. TS 6.9.1.6.2 requires that the analytical methods used to determine the core operating limits have been previously reviewed and approved by the NRC at the time the reload analyses are performed, and that the approved revision number is identified in the COLR.

3.0 EVALUATION

The licensee has proposed revising the current TS 6.9.1.6.2 list of analytical methodology references used to determine core operating limits. Specifically, the references listed in TS 6.9.1.6.2.e related to the steam line break (SLB) methodology and TS 6.9.1.6.2.f related to large break loss-of-coolant accident (LBLOCA) methodology would be changed. In addition, a new reference to a core physics computer code system for pressurized water reactors would be added as TS 6.9.1.6.2.o.

TS 6.9.1.6.2.e

TS 6.9.1.6.2.e lists the references which define the SLB methodologies. These methodologies are used in developing the core operating limits for TS 3.1.1.3 - Moderator Temperature

Coefficient, TS 3.1.3.5 - Shutdown Bank Insertion Limits, TS 3.1.3.6 - Control Bank Insertion Limits, and TS 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.

Currently, TS 6.9.1.6.2.e lists the reference:

XN-NF-84-93(A), latest Revision and Supplements, "Steamline Break Methodology for PWRs," Exxon Nuclear Company, Richland WA 99352.

The license proposes deleting the above reference and adding the references:

EMF-84-093(P)(A), "Steam Line Break Methodology for PWRs," approved version as specified in the COLR.

ANF-84-093(P)(A), "Steam Line Break Methodology for PWRs," approved version as specified in the COLR.

Since the SLB methodology references proposed by the licensee have been approved by the NRC, and are applicable to HNP, the staff finds the proposed change acceptable.

TS 6.9.1.2.f

TS 6.9.1.6.2.f lists the references which define the LBLOCA evaluation model. This model is used in developing the core operating limits for TS 3.2.1 - Axial Flux Difference, TS 3.2.2 - Heat Flux Hot Channel Factor, and TS 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor.

Currently, TS 6.9.1.6.2.f references:

EXEM PWR Large Break LOCA Evaluation Model as defined by:

XN-NF-82-20(A), latest Revision and Supplements, "Exxon Nuclear Company Evaluation Model EXEM/PWR ECCS Model Updates," Exxon Nuclear Company, Richland WA 99352.

XN-NF-82-07(A), latest Revision, "Exxon Nuclear Company ECCS Cladding Swelling and Rupture Model," Exxon Nuclear Company, Richland WA 99352.

XN-NF-81-58(A), latest Revision and Supplements, "RODEX2 Fuel Rod Thermal Response Evaluation Model," Exxon Nuclear Company, Richland WA 99352.

XN-NF-85-16(A), Volume 1 and Supplements, Volume 2, latest Revision and Supplements, "PWR 17x17 Fuel Cooling Test Program," Exxon Nuclear Company, Richland WA 99352.

XN-NF-85-105(A), and Supplements, "Scaling of FCTF Based Reflood Heat Transfer Correlation for Other Bundle Designs," Exxon Nuclear Company, Richland WA 99352.

The license proposes deleting the above references and adding the references:

SEM/PWR Large Break LOCA Evaluation Model as defined by:

EMF-2087(P)(A), "SEM/PWR-98: ECCS Evaluation Model for PWR LBLOCA Applications," approved version as specified in the COLR.

XN-NF-81-58(P)(A), "RODEX2 Fuel Rod Thermal-Mechanical Response Evaluation Model," approved version as specified in the COLR.

ANF-81-58(P)(A), "RODEX2 Fuel Rod Thermal Mechanical Response Evaluation Model," approved version as specified in the COLR.

XN-NF-82-06(P)(A), "Qualification of Exxon Nuclear Fuel for Extended Burnup," approved version as specified in the COLR.

ANF-88-133(P)(A), "Qualification of Advanced Nuclear Fuels' PWR Design Methodology for Rod Burnups of 62 GWd/MTU," approved version as specified in the COLR.

XN-NF-85-92(P)(A), "Exxon Nuclear Uranium Dioxide/Gadolinia Irradiation Examination and Thermal Conductivity Results," approved version as specified in the COLR.

EMF-92-116(P)(A), "Generic Mechanical Design Criteria for PWR Fuel Designs," approved version as specified in the COLR.

Since the LBLOCA methodology references proposed by the licensee have been approved by the NRC, and are applicable to HNP, the staff finds the proposed change acceptable.

TS 6.9.1.6.2.o

The licensee proposes adding a new reference to a core physics computer code system for pressurized water reactors as TS 6.9.1.6.2.o. This reference is used in the development of the core operating limits for TS 3.1.1.2 - Shutdown Margin - Modes 3, 4 and 5, TS 3.1.1.3 - Moderator Temperature Coefficient, TS 3.1.3.5 - Shutdown Bank Insertion Limits, TS 3.1.3.6 - Control Bank Insertion Limits, TS 3.2.1 - Axial Flux Difference, TS 3.2.2 - Heat Flux Hot Channel Factor, TS 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor, and TS 3.9.1 - Boron Concentration.

The proposed reference is:

EMF-96-029(P)(A), "Reactor Analysis Systems for PWRs," approved version as specified in the COLR.

Since the methodology reference proposed by the licensee has been approved by the NRC, and is applicable to HNP, the staff finds the proposed change acceptable.

Administrative changes

In addition to the changes in the references discussed above, the licensee proposed changing the format of other cited references in TS 6.9.1.6.2 to maintain consistency within the TS. The

licensee proposed changing references that cited the "latest Revision," to state "approved version as specified in the COLR" instead. This change will allow the licensee to use current topical reports to support limits in the COLR without having to submit a license amendment every time the topical report is revised. The COLR will provide specific information identifying the particular approved topical reports used to determine the core limits for the particular cycle in the COLR report. The staff, therefore, finds this change acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the State of North Carolina official was notified of the proposed issuance of the amendment. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. Accordingly, this amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need to be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributor: R. Laufer

Date: February 10, 2000

Mr. James Scarola
Carolina Power & Light Company

Shearon Harris Nuclear Power Plant
Unit 1

cc:

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AMENDMENT NO. 94 TO FACILITY OPERATING LICENSE NO. NPF-63 - HARRIS, UNIT 1

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PDII Reading

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