

January 20, 2000

Mr. Harold W. Keiser
Chief Nuclear Officer & President -
Nuclear Business Unit
Public Service Electric & Gas
Company
Post Office Box 236
Hancocks Bridge, NJ 08038

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON CONTAINMENT INSERVICE
INSPECTION RELIEF REQUESTS FOR HOPE CREEK GENERATING STATION
AND SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 (TAC NOS.
MA6865, MA6866, AND MA6867)

Dear Mr. Keiser:

By letter dated October 7, 1999, the Public Service Electric and Gas Company (PSE&G) submitted relief requests for the containment inservice inspection program for the Hope Creek and Salem Generating Stations. Specifically, PSE&G requested relief from the requirements of Subsection IWE and IWL of the 1992 Edition and Addenda of Section XI of the American Society of Mechanical Engineers' Boiler and Pressure Vessel Code (the Code) and proposed that the 1998 Edition of the Code be used as an alternative for those subsections.

The Nuclear Regulatory Commission staff, with assistance of the Idaho National Engineering and Environmental Laboratory, is reviewing your submittal and has determined that additional information is required to complete the review. The specific information requested is addressed in the enclosure. This information is necessary to complete our review, which is required by 10 CFR 50.55a(a)(3)(i). We request that the additional information be provided within 30 days of receipt of this letter, as agreed with Mr. James Priest of your staff on December 29, 1999. If circumstances require you to revise your response date, or if you have any questions, please call me at (301) 415-1479.

Sincerely,

/RA/

William C. Gleaves, Project Manager, Section 2
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-354, 50-272,
and 50-311

Enclosure: Request for Additional Information

cc w/encl: See next page

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REQUEST FOR ADDITIONAL INFORMATION

CONTAINMENT INSERVICE INSPECTION

RELIEF REQUESTS RR-E1 AND RR-L1

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

HOPE CREEK GENERATING STATION, UNIT 1

By letter dated October 7, 1999, Public Service Electric and Gas Company (PSE&G), submitted relief requests RR-E1 and RR-L1 for the containment inservice inspection program for Salem Nuclear Generating Station, Unit Nos. 1 and 2, and Hope Creek Generating Station, Unit 1. The relief requests contain proposed alternatives to use the 1998 Edition of the American Society of Mechanical Engineers' (ASME) Boiler and Pressure Vessel Code, Section XI, in lieu of the 1992 Edition and Addenda for Subsections IWE and IWL inspections.

The Nuclear Regulatory Commission, with assistance from the Idaho National Engineering and Environmental Laboratory, has reviewed all information provided by the licensee in Relief Requests RR-E1 and RR-L1. The staff has determined that the following additional information is necessary to complete our review, which is required by 10 CFR 50.55a(a)(3)(i).

1. The 1998 Code Edition, Subsections IWE and IWL, defer to *owner-defined* General and Detailed visual examinations in lieu of accepted visual examination requirements, as currently described in IWA-2000. Please describe the following attributes of your visual examination program, addressing both IWE and IWL components:
 - Describe the General visual acceptance criteria that will be used to evaluate indications found on containment surfaces, containment welds, bolting, moisture barriers, dissimilar metal welds, etc.
 - Each relief request states that Detailed and General visual examinations are either similar or equivalent to existing visual examinations (VT-1 and VT-3). However, it is unclear whether PSE&G is committing to use VT-1 and VT-3 requirements and acceptance criteria in lieu of the owner-defined General and Detailed examinations found in Article IWE and IWL. Please clarify if the General and Detailed visual examinations incorporate all existing VT-1 and VT-3 examination requirements. In addition, state whether personnel performing these examinations will be VT-1 or VT-3 qualified. If existing VT-1 or VT-3 requirements will not be used, describe all General and Detailed visual criteria used to address visual examinations.
2. The 1992 Edition of the Code requires a visual examination (VT-1) of bolting when a connection is disassembled. The 1998 Edition requires a General visual,

Enclosure

performed in place, with no requirement for visual examination when the joint is disassembled. It is not clear what, if any, examinations will be performed on disassembled bolted connections. If VT-1 examinations will not be performed on this type of connection, please provide an explanation and basis for how this practice provides an acceptable level of quality and safety.

3. IWL-2410 allows for deferral of concrete visual exams to the next scheduled plant outage for portions of the concrete surface which cannot be examined within the stated time interval. Please explain if credit will be taken for two successive intervals.