



UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
SAM NUNN ATLANTA FEDERAL CENTER
61 FORSYTH STREET, SW, SUITE 23T85
ATLANTA, GEORGIA 30303-8931

February 7, 2000

Florida Power and Light Company
ATTN: Mr. T. F. Plunkett
President - Nuclear Division
P. O. Box 14000
Juno Beach, FL 33408-0420

SUBJECT: NRC INTEGRATED INSPECTION REPORT 50-250/99-08, 50-251/99-08

Dear Mr. Plunkett:

This refers to the inspection conducted on November 28, 1999 through January 8, 2000, at the Turkey Point 3 and 4 reactor facilities. The enclosed report presents the results of that inspection.

During the inspection period, your conduct of activities was generally characterized by safety conscious operations. Maintenance activities were well coordinated and planned.

Based on the results of the inspection, the NRC has determined that one violation of NRC requirements occurred. This violation is being treated as a Non-Cited Violation (NCV), consistent with Section VII.B.1.a of the NRC Enforcement Policy. The NCV is described in the subject inspection report. If you contest the violation or severity level of the NCV, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington DC 20555-0001, with copies to the Regional Administrator, Region II, the Resident Inspector at your facility and the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington DC 20555-0001.

In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be placed in the NRC Public Document Room (PDR).

Sincerely,

/R/

Leonard D. Wert, Chief
Reactor Projects Branch 3
Division of Reactor Projects

Docket Nos. 50-250, 50-251
License Nos. DPR-31, DPR-41

Enclosure: Inspection Report Nos. 50-250/99-08, 50-251/99-08

cc w/encl: (See page 2)

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U.S. NUCLEAR REGULATORY COMMISSION

REGION II

Docket Nos: 50-250, 50-251
License Nos: DPR-31, DPR-41

Report Nos: 50-250/99-08, 50-251/99-08

Licensee: Florida Power and Light Company

Facility: Turkey Point Nuclear Plant, Units 3 & 4

Location: 9760 S. W. 344 Street
Florida City, FL 33035

Dates: November 28, 1999 - January 08, 2000

Inspectors: C. Patterson, Senior Resident Inspector
R. Reyes, Resident Inspector
G. Kuzo, Regional Inspector (Section R1.1, R1.2, R2.1, R3.1,
R7.1, R7.2)

Approved by: L. Wert, Chief
Reactor Projects Branch 3
Division of Reactor Projects

Enclosure

EXECUTIVE SUMMARY

Turkey Point Nuclear Plant, Units 3 & 4 NRC Inspection Report 50-250/99-08, 50-251/99-08

This integrated inspection included aspects of licensee operations, maintenance, engineering, and plant support. The report covers a 6-week period of resident inspection; in addition, it includes the results of inspections by a regional radiation specialist.

Operations

- The licensee was satisfactorily implementing the cold weather procedural requirements. The procedure addressed Technical Specification requirements and equipment vulnerable in a cold weather environment (Section O1.2).

Maintenance

- The 3A emergency diesel generator radiator replacement was well coordinated and planned. The compensatory measures required by the one time technical specification amendment to extend the allowed outage time were properly implemented (Section M1.1).

Engineering

- The meteorological systems were well maintained and routinely checked by walkdown and inspection procedures (Section E2.1).
- The vital direct current electrical system was well maintained. Maintenance on the 4B station batteries was satisfactorily completed. The system alignment, material condition, and housekeeping supported plant operations (Section E2.2).

Plant Support

- Radiological controls and monitoring for low-level radioactive waste processing/storage areas and effluent processing activities were implemented effectively and maintained in accordance with Updated Final Safety Analysis Report, Technical Specifications, and 10 CFR Part 20 requirements. Occupational worker doses were within administrative and regulatory limits. The onsite storage and disposal of solid radioactive waste was managed effectively (Section R1.1).
- Processing, packaging, and shipment of radioactive waste for disposal met 10 CFR Parts 20, 61, and 71, and 49 CFR Parts 170-189 requirements (Section R1.2).
- Maintenance workers demonstrated limited knowledge of procedural steps and equipment requirements for the initial movement and packaging of a shielded high integrity container of spent resins in preparation for transport on December 1, 1999 (Section R1.2).
- The observed radiation monitoring system equipment installation, and detector electronic and source calibrations met regulatory requirements (Section R2.1).

- The 1998 Annual Effluent Release Report and Annual Radiological Environmental Monitoring Report were submitted in accordance with TS and documented results demonstrated gaseous and liquid effluent processing and subsequent releases met established regulatory limits and did not significantly impact the surrounding environs (Section R3.1).
- Licensee actions for multiple radiation protection condition report issues associated with high radiation areas were appropriate and completed in a timely manner. A non-cited violation was identified for failure to maintain or establish high radiation area postings (Section R7.1).
- Counting-room Quality Control activities were implemented effectively and verified the accuracy of radionuclide analytical measurement instrumentation (Section R7.2).
- Physical security activities at the Nuclear Entrance Building, and operability tests on security search equipment, were satisfactorily performed. Security personnel were very knowledgeable with the test requirements and acceptance criteria (Section S1.1).

Report Details

Summary of Plant Status

Both units operated at full power this period. Unit 3 has been online since on November 18, 1999. Unit 4 has been online since April 18, 1999.

I. Operations

O1 Conduct of Operations

O1.1 General Comments (71707)

Using Inspection Procedure 71707, the inspectors conducted frequent reviews of ongoing plant operations. In general, the conduct of operations was professional and safety-conscious; specific events and noteworthy observations are detailed in the sections below.

O1.2 Cold Weather Operations

a. Inspection Scope (71707)

The inspectors reviewed the licensee's procedures and readiness for plant operations in a cold weather environment.

b. Observations and Findings

The licensee enters Off Normal Operating Procedure 0-ONOP-103.2, Cold/Hot Weather Conditions, if any of the following cold weather conditions are satisfied: auxiliary building temperature less than 65 degrees Fahrenheit (°F); actual outside air temperature less than 55°F; or if outside air temperature is predicted to go below 32°F.

Through a detailed review of the procedure, and several discussions with system engineers and Operations personnel, the inspectors verified that the licensee addressed Technical Specifications (TS) requirements, and equipment vulnerable in a cold weather environment, i.e., boric acid storage tank room, charging pump rooms, and refueling water storage tank solution temperatures. For example, if auxiliary building ambient temperature decreased to 65°F, the licensee uses 15 kilowatt heaters at various locations to maintain room temperatures above TS limits. The basis for the 15 kilowatt heaters, as described in Calculation PTN-BFJM-90-041, Auxiliary Building Heating For B.A. Concentration Reduction, was reviewed with the responsible system engineer.

The temperature recorder located in the auxiliary building continuously monitors the boric acid storage tank room temperature and three other locations in the building. Also, the recorder initiates a control room alarm if any temperature falls below a specified limit. The inspectors reviewed the preventive maintenance procedure used to calibrate the recorder and annunciator with maintenance supervisors. Additionally, through review of work control documentation, the inspectors verified that the calibration and maintenance were being performed per the prescribed frequency.

The inspectors reviewed operating procedures 0-OP-025, Control Room Ventilation System (CRVS); and 4-OP-079.1, Unit 4 EDG Building Heating Ventilation and Air Conditioning (HVAC). The CRVS procedure described that during periods when the outdoor temperature is less than 65°F, if the control room thermostats are set to less than 72°F, the evaporator coils of the air handler may freeze up. This may result in the loss of cooling capability of the control room HVAC system. Setting the EDG building temperature controls lower than 79°F could cause the EDG HVAC units to ice during cold weather periods. The inspectors noted that the cold weather procedure did not specifically address the temperature control settings. The licensee initiated a condition report to address the issue.

Through review of the control room logs, the inspectors verified the licensee had entered the cold weather procedure on several occasions through the inspection period when outside ambient temperature had fallen below 55°F. The inspectors reviewed the applicable Operations Tour Procedures used by the non-licensed operators for obtaining temperatures in the auxiliary building and outside ambient. Additionally, the inspectors, accompanied a non-licensed operator during one of the outside tours and verified that temperatures were being recorded as described in the procedure.

c. Conclusions

The licensee was satisfactorily implementing the cold weather procedural requirements. The procedure addressed Technical Specification requirements and equipment vulnerable in a cold weather environment.

O1.3 Clearance Verification (71707)

On January 4, 2000, the inspectors verified that clearance order 3-00-01-07 was adequate for cleaning of the 3C Component Cooling Water (CCW) heat exchanger. The clearance boundary was verified using the system drawing. A boundary modification had been made to the original clearance due to a broken vent valve and an alternate vent valve was tagged open. The placement of the clearance tags and proper valve position was verified at the heat exchanger. No deficiencies were noted. The proper operational alignment of the other two CCW heat exchangers was also verified.

O1.4 Year 2000 Rollover Date (71707)

The inspectors monitored the licensee's preparation for the year 2000 critical date rollover. The inspectors attended several preparation meetings, pre-job briefing, and observed control room activities during the rollover. The licensee's activities were formalized under procedure TP-099-037, Year 2000 Critical Date Rollover Operating and Contingency Procedure. There was upper management on-site in the control room with support staff on-site for any potential problems. The procedure was accomplished as planned and there were no issues with the rollover to January 1, 2000.

II. Maintenance

M1 Conduct of Maintenance

M1.1 Maintenance Work Order and Surveillance Observations

a. Inspection Scope (61726) (62707)

The inspectors observed the following surveillance and maintenance activities:

| | |
|---------------|--|
| 3-OP-030 | 3C Component Cooling Water Heat Exchanger Cleaning |
| WO99010935 | A" Service Water Pump |
| WO29002831-01 | Change out 4B Station Batteries |
| 4-OSP-019.1 | Intake Cooling Water Inservice Test |
| WO98010395 | Fire Door Replacement Between Cable Spreading Room and Air Conditioning Fan Room |
| WO28020262 | 3A Emergency Diesel (EDG) Radiator Replacement |

b. Observations and Findings

On December 13, 1999, the 3A EDG was taken out of service to replace the radiator. A one-time extension of the EDG allowed outage time from 72 hours to seven days was granted by a Technical Specification amendment dated November 19, 1999. Three compensatory measures were applied to accomplish this work. These were: (1) schedule the replacement outside hurricane season and when no adverse weather was expected, (2) postpone the performance of any load threatening surveillance tests until the EDG was returned to service, and (3) control personnel access to the switchyard.

This work activity was well planned and prepared by the licensee. The compensatory measures were discussed in the plan of the day schedule and operations night orders and other site meetings. The inspectors monitored the work activities during the radiator replacement. The inspectors verified that the compensatory measures were established during the radiator replacement. The licensee strictly controlled access to the operable 3B EDG including posting signs at the entrance doors. Daily observations of the switchyard were made to confirm no switchyard activities. The hurricane season ended in November and there was no severe weather during the radiator replacement. Some difficulty was encountered with gasket material but the assistance of a radiator vendor was obtained and the issue was satisfactorily resolved. The 3A EDG was returned to service on December 18, 1999.

c. Conclusions

The 3A emergency diesel generator radiator replacement was well coordinated and planned. The compensatory measures required by the one time technical specification amendment to extend the allowed outage time were properly implemented.

III. Engineering

E2 Engineering Support of Facilities and Equipment

E2.1 Meteorological System

a. Inspection Scope (37551) (71750)

On December 14, 1999, the inspectors performed an inspection of the meteorological system with the system engineer.

b. Observations and Findings

The inspectors performed a visual inspection of the 10 meter and 60 meter meteorological towers. These towers provide wind speed and direction information. The tower locations are specified in TS Figure 5.1 - 1. Each tower was secure with guide wires. Each support shelter and electronic rooms were well maintained. The electronics room was maintained cool by window air conditioners. The electronics equipment provides indication to the control room of a rolling 15 minute average of wind speed. Each backup electrical generator room in the shelter was clean and free of fire hazards. The 60 meter tower shelter which is the primary site, had redundant backup electrical generators and electronic room air conditioners.

The 10 and 60 meter towers are designed for 100 mile per hour (mph). However, located next to each tower was a high wind monitoring station designed for 250 mph. These were added after Hurricane Andrew. Although currently not connected to the control room, consideration was being given to make this information available to the control room. All of the equipment was checked by routine inspections. The inspectors reviewed procedure Met - Dir - 001, Meteorological System Walkdown and Inspections. The system engineer was very knowledgeable on all aspects of this equipment.

c. Conclusions

The meteorological systems were well maintained and routinely checked by walkdown and inspection procedures.

E2.2 Vital Direct Current (DC) System

a. Inspection Scope (37551) (71707)

The inspectors performed a general walkdown of the vital DC system.

b. Observations and Findings

The inspectors performed a general walkdown of the DC equipment which consisted of the busses, batteries, battery chargers and the inverters. Also, the inspectors walked down the system with the responsible system engineer. The battery replacement work that was in progress on the 4B station batteries was reviewed with engineering and maintenance personnel in the field. Additionally, entrance into an action statement as a result of the battery maintenance was also reviewed. A recently completed station battery weekly surveillance procedure was reviewed for completeness and TS compliance. Issues and relevant maintenance history of the vital batteries, and Operating Experience information were reviewed with the chief electrical engineer from Corporate Engineering. The inspectors identified no concerns.

c. Conclusions

The vital direct current electrical system was well maintained. Maintenance on the 4B station batteries was satisfactorily completed. The system alignment, material condition, and housekeeping supported plant operations.

IV. Plant Support

R1 Radiological Protection and Chemistry Controls

R1.1 Conduct of Radiological Protection Controls

a. Inspection Scope (84750, 86750)

Radiological controls associated with radioactive effluent processing and release, solid radioactive waste processing/storage, and radioactive material storage areas were reviewed. During tours of radiologically controlled areas (RCA), the inspectors evaluated general housekeeping, radiological monitoring, postings, and physical controls for high radiation areas (HRAs) and locked-high radiation areas (LHRAs) associated with radioactive effluent, material, and solid waste processing and storage.

Procedures and established radiological controls were compared against applicable sections of the Updated Final Safety Analysis Report (UFSAR), Technical Specifications (TS), and 10 CFR Part 20 and Part 50 requirements.

b. Observations and Findings

Radiological housekeeping practices and surface contamination levels within the auxiliary building and radioactive waste/material processing and storage areas were acceptable, with RCA unrestricted areas maintained radiologically clean and uncluttered. Area postings and container labels were maintained in accordance with the associated access controls or radiological conditions. High and locked-high radiation areas were controlled appropriately. Radioactive waste inventories were monitored and controlled to established goals. No concerns were identified for solid radioactive waste storage areas.

Doses for occupational workers involved in radioactive waste processing and effluent release activities were within regulatory requirements. As of December 2, 1999, the maximum year-to-date individual total effective dose equivalent reported was 848 millirem.

c. Conclusions

Radiological controls and monitoring for low-level radioactive waste processing/storage areas and effluent processing activities were implemented effectively and maintained in accordance with Updated Final Safety Analysis Report, Technical Specifications, and 10 CFR Part 20 requirements. Occupational worker doses were within administrative and regulatory limits. The onsite storage and disposal of solid radioactive waste was managed effectively.

R1.2 Radioactive Waste and Material Transportation Activities

a. Inspection Scope (86750)

Radiation protection (RP) program activities associated with characterization, packaging, and transportation of radioactive waste and subsequent burial were reviewed. For selected shipments made between June 1, through December 3, 1999, implementing procedures, quality control records, and shipping paper and supporting documentation were reviewed and evaluated for accuracy and completeness. Direct observations and evaluation of radiation controls and maintenance activities for preparation and packaging of a high integrity container filled with spent resins for subsequent shipment were conducted on December 1-2, 1999.

Program guidance and implementation were evaluated against 10 CFR Parts 20, 61, and 71, and Department of Transportation (DOT) 49 CFR Parts 170-189 regulations.

b. Observations and Findings

Licensee procedural guidance met applicable regulatory requirements. The 10 CFR Part 61 radionuclide analysis frequency exceeded guidance specified in the Final Waste Classification and Waste Form Technical Position Paper dated May 1983. For the radioactive waste shipments reviewed, documentation was accurate and complete.

The inspectors noted several performance issues associated with initial mockup training, and preparation for the December 1, 1999 movement, packaging, and preparation of a shielded-liner containing spent resins for subsequent transport. The initial mock-up training verified clearance requirements for the shielded container but did not exercise the remote operations required by the crane operator and other maintenance workers. Further, the maintenance workers demonstrated limited understanding of the procedural steps and equipment requirements for packaging activities. Subsequent to completion of additional mockup training, procedural reviews, equipment calibrations, and a second "As Low As Reasonably Achievable" briefing, the task was completed successfully on December 2, 1999.

c. Conclusions

Processing, packaging, and shipment of radioactive waste for disposal met 10 CFR Parts 20, 61, and 71, and 49 CFR Parts 170-189 requirements.

Maintenance workers demonstrated limited knowledge of procedural steps and equipment requirements for the initial movement and packaging of a shielded high integrity container of spent resins in preparation for transport on December 1, 1999.

R2 Status of Radiological Protection and Chemistry Equipment

R2.1 Radiation Monitor System (RMS) Installation and Calibrations

a. Inspection Scope (84750)

The inspectors reviewed and evaluated installed process and effluent Radiation Monitoring System (RMS) detectors, sampling lines and flow meters to implement Offsite Dose Calculation Manual (ODCM) and 10 CFR Part 20 requirements. The evaluation included, as applicable, Unit 3 and 4 RMS equipment walk-downs with comparisons against UFSAR commitments, design specifications, and recommendations detailed in American National Standards Institute N13.1-1969, American National Standard Guide to Sampling Airborne Radioactive Materials in Nuclear Facilities.

Approved calibration guidance and results for the Unit 3 Plant Vent RMS, the Control Room HVAC Detectors, and Containment High Range Monitors (CHRMs) were reviewed and discussed. System calibrations were reviewed and evaluated against applicable sections UFSAR sections, and TS and ODCM requirements. For the CHRMs, special calibrations to meet Three Mile Island (TMI) Action Item II.F.1 were reviewed.

b. Observations and Findings

The RMS detectors and sampling lines were installed in accordance with UFSAR descriptions and established industry practices. Responsible staff were knowledgeable of system design changes, operational requirements and performance trends. Monitor

calibrations were conducted at the required frequencies and final results were within established acceptance criteria.

c. Conclusions

The observed radiation monitoring system equipment installation, and detector electronic and source calibrations met regulatory requirements.

R3 Radiological Protection and Chemistry Documents

R3.1 Radiological Effluent and Environmental Monitoring Reports

a. Inspection Scope (84750)

The inspector reviewed and discussed the 1998 Radioactive Effluent Release Report and the 1998 Annual Radiological Environmental Monitoring Program Report submitted in accordance with TS 6.9.1.4 and 6.9.1.3, respectively.

b. Observations and Findings

For 1998, liquid and gaseous effluent radionuclide concentrations were significantly less than 10 CFR Part 20 limits, and projected doses were significantly below ODCM limits and design objectives listed in Appendix I to 10 CFR Part 50. The report did not identify significant changes to radioactive waste treatment systems, or new dose calculations as a result of the land-use census.

The Radiological Environmental Monitoring Report did not identify any long-term increasing trends in radionuclide concentrations for media sampled with the majority of results reported as less than the specified lower limit of detection.

c. Conclusions

The 1998 Annual Effluent Release Report and Annual Radiological Environmental Monitoring Report were submitted in accordance with TS and documented results demonstrated gaseous and liquid effluent processing and subsequent releases met established regulatory limits and did not significantly impact the surrounding environs.

R7 Quality Assurance in Radiation Protection and Chemistry Activities

R7.1 Radiation Protection and Chemistry Condition Reports

a. Inspection Scope (83750, 84750, 86750)

The inspectors reviewed details and status of five Condition Reports (CR) documented between October 8, through November 26, 1999, for potential high radiation and potential locked high radiation area control issues associated with radioactive waste processing/storage and release activities. Identified issues and licensee actions were evaluated against TS, and 10 CFR Parts 20 and 50.

b. Observations and Findings

From discussion and review of followup radiation surveys and subsequent evaluations, the inspectors determined that none of the instances documented by the licensee involved actual locked-high radiation area conditions nor resulted in any significant unanticipated exposures to workers. 10 CFR 20.1902 (b) states that the licensee shall post each high radiation area with a conspicuous sign or signs bearing the radiation symbol and the words "CAUTION, HIGH RADIATION AREA" or "DANGER, HIGH RADIATION AREA." For three of the documented CRs, the inspectors determined that the failure to maintain or establish high radiation postings/barriers for changing radiological conditions was in violation of 10 CFR 20.1902 (b) requirements. Areas with radiation levels above 100 millirem per hour were not posted. In one case, the postings and barricade to the auxiliary building 10 foot elevation pipeway entrance door were improperly removed. Subsequent surveys identified the location as a high radiation area. In the other two instances, plant conditions were changed and the resulting high radiation areas were not posted for several days. For the identified actual high radiation issues, the completed actions were prioritized, technically correct, and long-term corrective actions were scheduled. These multiple examples of a Severity Level IV violation are being treated as a non-cited violation, consistent with Section VII.B.1.a of the NRC Enforcement Policy, is identified as Non-cited Violation 50-250, 251/99-08-01, Failure to Maintain or Establish High Radiation Area Postings. This violation is in the licensee's corrective action program as CRs 99-1456, 99-1458, and 99-1474.

c. Conclusions

Licensee actions for multiple radiation protection condition report issues associated with high radiation areas were appropriate and completed in a timely manner. A non-cited violation was identified for failure to maintain or establish high radiation area postings.

R7.2 Effluent Release Quality Control (QC) Activities (84750)

The inspectors reviewed implementation of the effluent monitoring program Quality Control (QC) activities to meet the intent of Regulatory Guide 4.15, Revision 1, February 1978. Reviewed activities included counting room analytical radionuclide measurement system QC performance activities.

b. Observations and Findings

The counting room instrumentation QC program was implemented in accordance with approved procedures. In-service gamma-spectroscopy systems' QC performance data were acceptable. Results of inter-laboratory crosscheck analyses were acceptable.

c. Conclusions

Counting-room Quality Control activities were implemented effectively and verified the accuracy of radionuclide analytical measurement instrumentation.

S1 Conduct of Security and Safeguards Activities**S1.1 Security Search Equipment Testing****a. Inspection Scope (71750)**

The inspectors observed physical security activities at the Nuclear Entrance Building, reviewed the security procedures on testing of Search Equipment, and observed the operational testing in progress.

b. Observations and Findings

On December 8, 1999, the inspectors observed routine physical security entrance activities at the Nuclear Entrance Building (NEB), and observed security guards perform Security Force Instruction SFI-2301, Search Equipment Testing. The entrance activities were well coordinated and no problems were identified. The SFI delineates the methods and requirements for the testing of search equipment. Operability tests for the explosive detector, metal detectors (portal and hand held) were observed. All three detectors tested satisfactorily as described in the procedure test criteria. Additionally, at the Truck Gate Entrance (TGE), the inspectors reviewed the testing criteria with a security guard stationed there. The guard was very knowledgeable with the procedure and test requirements. The inspector verified that explosive detector testing samples were labeled with the expected 90 day period expiration dates in accordance with procedural requirements.

c. Conclusions

Physical security activities at the Nuclear Entrance Building, and operability tests on security search equipment, were satisfactorily performed. Security personnel were very knowledgeable with the test requirements and acceptance criteria.

V. Management Meetings and Other Areas**X1 Exit Meeting Summary**

The inspectors presented the inspection results to members of licensee management at the conclusion of the inspection on January 12, 2000. An interim exit meeting was held on December 3, 1999, to discuss the findings of a Region-based inspection. The licensee acknowledged the findings presented.

The inspectors asked the licensee whether any materials examined during the inspection should be considered proprietary. No proprietary information was identified.

PARTIAL LIST OF PERSONS CONTACTED

Licensee

- D. Lowens, Quality Assurance Manager
- S. Franzone, Licensing Manager
- R. Hovey, Site Vice-President
- D. Jernigan, Plant General Manager
- T. Jones, Operations Manager
- J. Kirkpatrick, Protection Services Manager
- M. Lacal, Training Manager
- G. Hollinger, Work Control Manager
- R. Rose, Maintenance Manager
- E. Thompson, License Renewal Project Manager
- D. Tomaszewski, Site Engineering Manager
- J. Trejo, Health Physics/Chemistry Supervisor
- A. Zielonka, System Engineering Manager

Other licensee employees contacted included office, operations, engineering, maintenance, chemistry/radiation, and corporate personnel.

INSPECTION PROCEDURES USED

- IP 37551: Onsite Engineering
- IP 61726: Surveillance Observations
- IP 62707: Maintenance Observations
- IP 71707: Plant Operations
- IP 71750: Plant Support Activities
- IP 83750: Occupational Radiation Exposure
- IP 84750: Radioactive Waste Treatment, and Effluent and Environmental Monitoring
- IP 86750: Solid Radioactive Waste Management and Transportation of Radioactive Materials

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

| | | |
|---------------------|-----|---|
| 50-250,251/99-98-01 | NCV | Failure to Maintain or Establish High Radiation Area Postings. (Section R7.1) |
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Closed

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|---------------------|-----|---|
| 50-250,251/99-98-01 | NCV | Failure to Maintain or Establish High Radiation Area Postings. (Section R7.1) |
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Discussed

None