



Northern States Power Company

Monticello Nuclear Generating Plant 2807 West County Road 75 Monticello, MN 55362

January 27, 2000

Tech Spec Section 6.7.A.6

US Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

> MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

Annual Report of Safety Relief Valve Challenges January 1 through December 31, 1999

In accordance with Monticello Technical Specification section 6.7.A.6, and NUREG 0737 item II.K.3.3, the following Annual Report of Safety and Relief Valve Failures and Challenges is provided:

Failures:

5/11/1999 The topworks on safety relief valve (SRV) no. RV-2-71C was removed from service because of a suspected small leak across the SRV's pilot stage seat. A 20°F increase in the SRV's discharge line was recorded following a plant scram on 5/8/99.

5/12/1999 The topworks on SRV no. RV-2-71G was removed from service because of a small through-wall leak in the SRV bellow's seal weld. This leak was discovered during normal operation by the bellows leak detection system. Subsequent investigation determined the source of the leak to be the seal weld.

Challenges:

5/8/1999 SRV no. RV-2-71H was opened twice automatically by the plant's Low-Low Set System at a reactor pressure of 1052 psig following a plant scram on 5/8/99.

5/8/1999 SRV no. RV-2-71G was opened manually at a reactor pressure of approximately 1030 psig following a plant scram on 5/8/99.

5/27/1999 SRV nos. RV-2-71C and RV-2-71G were opened manually once each at a reactor pressure of approximately 150 psig in accordance with a post maintenance test performed during plant startup.

This letter contains no new NRC commitments, nor does it modify any prior commitments. Please contact Sam Shirey at (612) 295-1449 if you require additional information concerning this report.

Byron D. Day

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CC: Director, Office of Nuclear Regulatory Research

> Regional Administrator-III, NRC NRR Project Manager, NRC Resident Inspector, NRC

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