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LCV-1409

Docket No.: 50-425

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D. C. 20555

Ladies and Gentlemen:

**VOGTLE ELECTRIC GENERATING PLANT
INSERVICE INSPECTION SUMMARY REPORT**

Southern Nuclear Operating Company (SNC) submits herein the "Owner's Report for Inservice Inspection (Form NIS-1)" for inservice inspection activities conducted at Vogtle Electric Generating Plant, Unit 2 (VEGP-2) during its Fall 1999 maintenance/refueling outage. Paragraph IWA-6230 of the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 1989 Edition, requires submittal of the enclosed report. The supporting documentation, e.g., examination records, equipment and material certifications, etc., from which the enclosed "Owner's Report for Inservice Inspection (Form NIS-1)" was compiled is available for review upon request at the plant site.

In addition, "Owner's Reports for Repair and Replacements (Form NIS-2)" are included in the enclosed report for repairs and/or replacements of piping and components that were documented and completed at VEGP-2 from the end of the previous maintenance/refueling outage (Spring 1998) through the Fall 1999 maintenance/refueling outage. Any attachments, e.g., code data reports, etc., that are referenced in the completed NIS-2 forms will be made available for review upon request at the plant site.

A discussion on the examination of steam generator tubes performed during the Fall 1999 maintenance/refueling outage is included in the enclosed VEGP-2 "Owner's Report for Inservice Inspection (Form NIS-1)." That discussion constitutes the report required by VEGP Technical Specification 5.6.10.b to be submitted to the NRC within 12 months following completion of the steam generator tube examinations. As a result, a separate submittal will not be made to the NRC on steam generator tubes examined during the subject outage.

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Several steam generator tubes required plugging as a result of the examinations performed during the Fall 1999 maintenance/refueling outage at VEGP-2. Pursuant to the requirements of VEGP Technical Specification 5.6.10.a, a report was submitted to the NRC within 15 days of the completion of the steam generator tubing examinations and identified the number of steam generator tubes that required plugging during the subject outage. SNC letter LCV-1399 dated November 1, 1999, transmitted the VEGP Technical Specification-required report to the NRC. Details on which tubes were plugged are included in the discussion provided in the enclosed "Owner's Report for Inservice Inspection."

In addition to components already identified in current requests for relief, seven Class 1 piping welds, one Class 2 equipment weld, and one Class 2 piping weld had a limited surface or volumetric examination coverage. The affected welds included the following:

Class 1 Piping:

21201-029-30 (4" pipe to valve),
21201-036-6 (12" pipe to valve),
21201-036-7 (12" valve to pipe),
21204-023-16 (6" valve to pipe),
21204-042-1 (6" valve to pipe),
21204-042-15 (6" pipe to tee), and
21204-043-1 (6" valve to pipe).

Class 2 Equipment:

21208-P6-002-W03 (Integrally welded pump support bracket).

Class 2 Piping:

21205-008-80 (8" pipe to valve).

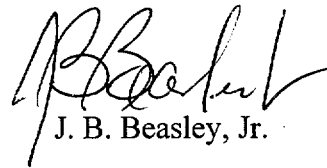
Physical limitations due to the geometric configuration of the welded area prevented full surface or ultrasonic examination coverage for each of the affected welds. As a result, the examination coverage was limited to less than ninety percent (<90%). ASME Section XI Code Case N-460, as approved by the NRC in NRC Regulatory Guide 1.147, allows Code acceptance when the reduction in examination coverage is less than ten percent (<10%). Because it is impractical to achieve the Code-required coverage for the affected welds due to the geometric configuration of the welded areas, request(s) for relief from the ASME Section XI Code requirements will be submitted to the NRC, as allowed by 10 CFR 50.55a, for review and approval. The applicable welds are indicated by "Relief Required" in the Class 1 piping, Class 2 equipment, and Class 2 piping weld tables found in the enclosed report.

In addition to the inservice inspection of Class 1, 2, and 3 components and supports during the VEGP-2 Fall 1999 maintenance/refueling outage, a containment examination was performed in response to the NRC rulemaking to 10 CFR 50.55a in which the requirements of Subsections IWE and IWL to the 1992 Edition of ASME Section XI with 1992 Addenda were invoked. A general visual examination of the VEGP-2 containment liner plate, a Class MC component as defined in Subsection IWE to the Code, was performed by SNC personnel during the outage. Anomalies, including arc strikes, discoloration, etc., were observed during the examination. Details on the general visual examination performed on the containment liner plate and the resolution of the indications that were observed are included in the discussion provided in the enclosed report.

Five copies of the "Owner's Report for Inservice Inspection (Form NIS-1)" for ISI activities conducted during the Fall 1999 maintenance/refueling outage at VEGP-2 are provided for your review. Two copies of the subject report are being provided to the Regional Administrator for NRC Region II.

Should there be any questions in this regard, please contact this office.

Sincerely,



J. B. Beasley, Jr.

JBB/JAE/

Enclosure: "Owner's Report for Inservice Inspection (Form NIS-1)" for VEGP-2
Seventh Maintenance/Refueling Outage (2R7), Fall 1999

xc: (see next page for distribution)

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