January 28, 2000

David J. Modeen, Director
Engineering, Nuclear Generation Division
Nuclear Energy Institute
1776 I Street, NW., Suite 400
Washington, DC 20006-3708

SUBJECT: STEAM GENERATOR DEGRADATION SPECIFIC MANAGEMENT DATABASE ADDENDUM 3

Dear Mr. Modeen:

By letter dated September 22, 1999, you submitted Addendum 3 to the Steam Generator Degradation Specific Management Database, as well as the associated correlations for use by the industry in alternate repair criteria applications, for NRC review and approval. By letter dated November 5, 1999, you withdrew the request for NRC approval of Addendum 3 for alternate repair criteria applications since the database changes did not result in a significant, non-conservative shift in the correlations. Your November 5, 1999, letter stated that the exclusion criteria were not revised, a revised probability of detection was not used, and questionable data was not used. Therefore, in accordance with the Steam Generator Degradation Specific Management Database Protocol that has been adopted by the NRC and industry, NRC approval is not required. The NRC staff concurs with that conclusion.

Your November 5, 1999, letter also noted that the industry is seeking NRC approval regarding four additional issues and requested that the NRC develop a timetable for completion of its review. The staff is focusing first on the two issues that will result in the most burden reduction, the tube pull program and voltage dependent probability of detection (POD). Regarding the Addendum 3 industry tube pull program proposal, the staff is currently reviewing the proposal and expects to complete its review by the requested January 31, 2000, target date. Regarding the recommendation that voltage dependent POD based on field experience be used in analyses supporting alternate repair criteria applications for outside diameter stress corrosion cracking at tube support plate intersections, the staff is currently reviewing the industry proposal and is targeting February 29, 2000, as a completion date for its review (one month later than requested).

For the remaining two issues, voltage dependent growth rate methodology and application of a bobbin to rotating pancake coil voltage correlation for dents greater than five volts, you requested approval by January 31 and September 5, 2000, respectively. The staff expects to complete its review of these two issues by August 2000. Based on discussions with your staff, we believe this will not impact any utilities.

Sincerely,

/ra/

Jack R. Strosnider, Director
Division of Engineering
Office of Nuclear Reactor Regulation

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Division of Engineering
Office of Nuclear Reactor Regulation

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