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JAN 13 2000

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station OP1-17
Washington, D.C. 20555

**SUSQUEHANNA STEAM ELECTRIC STATION
PROPOSED AMENDMENT NO. 228 TO LICENSE
NPF-14: REVISION TO THE LEGEND OF
TECHNICAL SPECIFICATION FIGURE 3.4.10.1
AND AMENDMENT NO. 191 TO LICENSE
NPF-22: REVISION TO THE LEGEND OF
TECHNICAL SPECIFICATION FIGURE 3.4.10.1
AND REVISE A REFERENCE IN TECHNICAL
SPECIFICATION SECTION 5.6.5.b
PLA-5148**

**Docket Nos. 50-387
and 50-388**

The purpose of this letter is to propose revisions to the Unit 1 and Unit 2 Susquehanna Steam Electric Station (SSES) Technical Specifications for NRC approval. These revisions revise the legend to Technical Specification Figure 3.4.10.1 "Reactor Vessel Pressure vs. Minimum Vessel Temperature" and correct a Unit 2 Core Operating Limits Report (COLR) reference listed in Technical Specification section 5.6.5.b. The proposed revisions are administrative and/or editorial in nature.

The proposed revision to the legend of Figure 3.4.10.1 will eliminate any confusion with respect to the intent of the legend and the technical basis for the Figure. The proposed Unit 2 revision to the reference in section 5.6.5.b corrects the supplement dates associated with the affected COLR reference.

Attachment 1 presents the Safety Assessment for the proposed revisions. This assessment concludes that these proposed administrative and/or editorial revisions to the SSES Unit 1 and Unit 2 Technical Specifications do not revise the Figure curves or the use of the Figure. Additionally, the corrections to the Unit 2 COLR reference does not change the methodology utilized to determine the core operating limits.

A001 Y1

Attachment 2 contains the “No Significant Hazards Consideration” and “Environmental Considerations” assessments. The “No Significant Hazards Considerations” assessment concludes that the proposed Technical Specification revisions do not involve a significant increase in the probability or consequence of an accident previously evaluated; do not create the possibility of a new or difference kind of accident from any accident previously evaluated; and do not involve a significant reduction in the margin of safety. The “Environmental Considerations” assessment concludes that the revisions conform to the criteria for actions eligible for categorical exclusion as specified in 10CFR51.22(c)(9), and will not impact the environment.

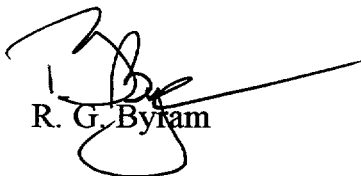
Attachment 3 contains marked-up pages of the Unit 1 and Unit 2 Technical Specifications. The bases associated with the proposed revisions are not affected by those revisions, and therefore, have not changed.

Attachment 4 contains “camera ready” versions of the revised Technical Specification pages.

The proposed changes have been approved by the SSES Plant Operations Review Committee and reviewed by the Susquehanna Review Committee.

Please contact Mr. R. D. Kichline at (610) 774-7705 if there are any questions concerning this submittal.

Sincerely,



R. G. Byram

Attachments

cc: NRC Region I
Mr. S. Hansell, NRC Sr. Resident Inspector
Mr. R. G. Schaaf, NRC Sr. Project Manager

**BEFORE THE
UNITED STATES NUCLEAR REGULATORY COMMISSION**

In the Matter of _____ :

PP&L, INC. :


Docket No. 50-387

**PROPOSED AMENDMENT NO. 228
FACILITY OPERATING LICENSE NO. NPF-14
SUSQUEHANNA STEAM ELECTRIC STATION
UNIT NO. 1**

Licensee, PP&L, Inc., hereby files proposed Amendment No. 228 to its Facility Operating License No. NPF-14 dated July 17, 1982.

This amendment contains a revision to the Susquehanna SES Unit 1 Technical Specifications.

PP&L, INC.
BY:



R. G. Byram
Sr. Vice-President
Generation and Chief Nuclear Officer

Sworn to and subscribed before me
this _____ day of _____, 2000.

Notarial Seal
Susan Grabowski-Turi, Notary Public
Allentown, Lehigh County
My Notary Commission Expires Sept. 8, 2003

**BEFORE THE
UNITED STATES NUCLEAR REGULATORY COMMISSION**

In the Matter of :

PP&L, INC. :

Docket No. 50-388

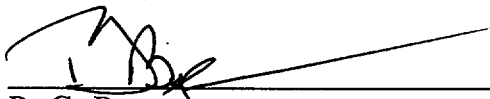
**PROPOSED AMENDMENT NO. 191
FACILITY OPERATING LICENSE NO. NPF-22
SUSQUEHANNA STEAM ELECTRIC STATION
UNIT NO. 2**

Licensee, PP&L, Inc., hereby files proposed Amendment No. 191 to its Facility Operating License No. NPF-22 dated March 23, 1984.

This amendment contains a revision to the Susquehanna SES Unit 2 Technical Specifications.

PP&L, INC.

BY:



R. G. Byram

Sr. Vice-President

Generation and Chief Nuclear Officer

Sworn to and subscribed before me
this _____ day of _____, 2000.

Notarial Seal
Susan Grabowski-Turi, Notary Public
Allentown, Lehigh County
My Commission Expires Sept. 8, 2003

Notary Public

ATTACHMENT 1 TO PLA-5148

SAFETY ASSESSMENT

SAFETY ASSESSMENT

I. DESCRIPTION OF PROPOSED CHANGE

This proposed revision to Technical Specification Figure 3.4.10.1, "Reactor Vessel Pressure vs. Minimum Vessel Temperature" is administrative in nature, and is applicable to both Unit 1 and Unit 2. The proposed revision is intended to clarify the legend on Figure 3.4.10.1 to broadly state the intent of the curves. Identical Pressure/Temperature (P/T) curves appear in both the Technical Specifications (Figure 3.4.10.1) and in the FSAR (Section 5.3). During the current licensing basis review of the FSAR it was determined that, while the P/T curves are technically correct, the legend information on the Figures in the Technical Specifications and the FSAR could be misleading and require clarification.

In the past the legends on the Figures in the FSAR were revised in an attempt to very briefly describe the technical basis of the curves. The legends on the Technical Specifications Figures were not revised. Consequently, the descriptions on the Figures in the FSAR and the Technical Specifications are not consistent and results in the same Figure in the Technical Specifications and the FSAR having different descriptions of the limiting non-beltline P/T curves.

The SSES P/T limiting non-beltline curves are composite curves established by superimposing stress analysis limits for those portions of the reactor pressure vessel (RPV) onto the most restrictive portions of the RPV (feedwater nozzles for the upper vessel region and CRD penetrations for the bottom head region). Development of the curves has evolved and now includes consideration of the highest nil ductility temperature among all of the vessel nozzles and other non-beltline component materials. This value is then applied to the appropriate feedwater nozzle or CRD penetration limit. The discussion of this technical basis methodology is explained in FSAR section 5.3.1.5.

The proposed revisions to section 5.6.5.b of the Unit 2 Technical Specifications (reference 4 to the Core Operating Limits Report (COLR)) are editorial in nature. The revision corrects the supplement dates to the existing referenced document. The current Technical Specifications only include the issuance date of the original supplement.

II. ANALYSIS

The proposed revision to the legend of Technical Specification Figure 3.4.10.1 does not impact the plant safety margins. The change is strictly administrative. The curves represented on the Figures remain unchanged. Additionally, the proposed amendment has no impact on the operator's use of the Figure, or on applicable operating procedures.

The proposed revision to section 5.6.5.b of the Unit 2 Technical Specifications (reference 4 to the COLR) are editorial in nature. No changes have been made to the methodology utilized to determine the core operating limits; therefore, there is no safety impact from the proposed revision.

III. CONCLUSIONS

Based on this safety assessment, it is concluded that the proposed revisions to the SSES Unit 1 and Unit 2 Technical Specifications Figure 3.4.10.1 and the revision to Unit 2 Technical Specification section 5.6.5.b are administrative and/or editorial; and therefore, do not create a change to the intent or use of the Technical Specifications.

ATTACHMENT 2 TO PLA-5148

**NO SIGNIFICANT HAZARDS CONSIDERATION
AND
ENVIRONMENTAL CONSIDERATION**

**NO SIGNIFICANT HAZARDS CONSIDERATIONS
AND
ENVIRONMENTAL CONSIDERATION**

NO SIGNIFICANT HAZARDS CONSIDERATIONS:

PP&L, Inc. has evaluated the proposed Technical Specification revisions in accordance with the criteria specified by 10CFR50.92 and has determined that the proposed revisions do not involve a significant hazards consideration. The criteria and conclusions of our evaluation are presented below.

- 1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.**

This proposal does not involve an increase in the probability or consequences of an accident previously evaluated. The proposed revision to Technical Specification Figure 3.4.10.1 and the proposed revision to the references in the Unit 2 Technical Specification section 5.6.5.b are administrative and/or editorial in nature, and do not involve a significant increase in the probability or consequences of an accident previously evaluated.

- 2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.**

This proposal does not create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed revision to Technical Specification Figure 3.4.10.1 and the proposed revision to the references in the Unit 2 Technical Specification section 5.6.5.b are administrative and/or editorial in nature. The proposed revisions do not change any plant systems, structures, or components, nor do they change any existing accident analysis, or create any new or different kind of accident from any accident previously evaluated.

- 3. The proposed change does not involve a significant reduction in the margin of safety.**

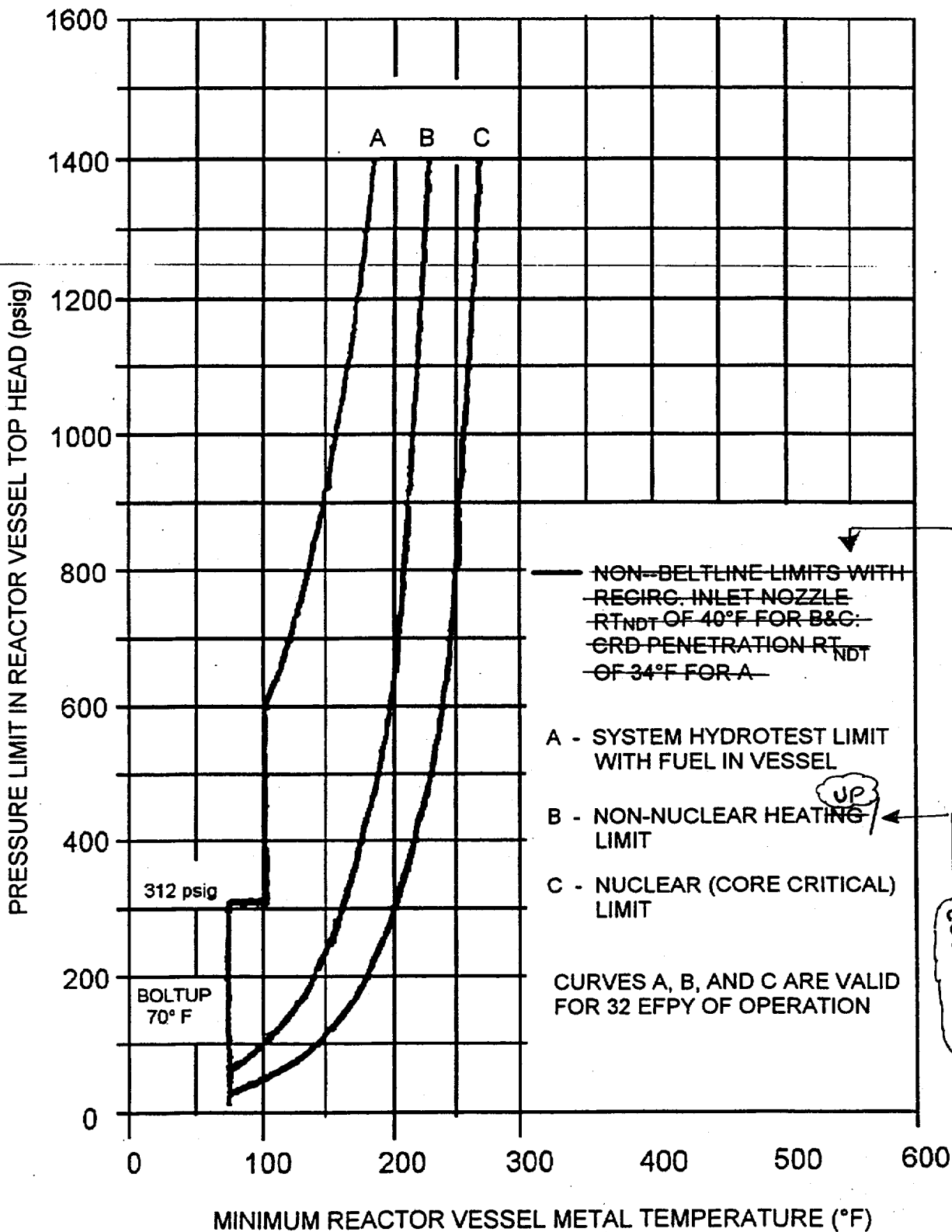
This proposal does not involve a significant reduction in the margin of safety. The proposed revision to Technical Specification Figure 3.4.10.1 and the proposed revision to the references in the Unit 2 Technical Specification section 5.6.5.b are administrative and/or editorial in nature, and do not result in significant reduction in the margin of safety.

ENVIRONMENTAL CONSIDERATION

An environmental assessment is not required for the proposed revisions because the requested revisions conform to the criteria for actions eligible for categorical exclusion as specified in 10CFR51.22(c)(9). The requested revisions will have no impact on the environment. As discussed above, the proposed revisions do not involve a significant hazard consideration. The proposed revisions do not involve a change in the types or increase in the amounts of effluents that may be released off-site. In addition, the proposed revisions do not involve an increase in the individual or cumulative occupational radiation exposure.

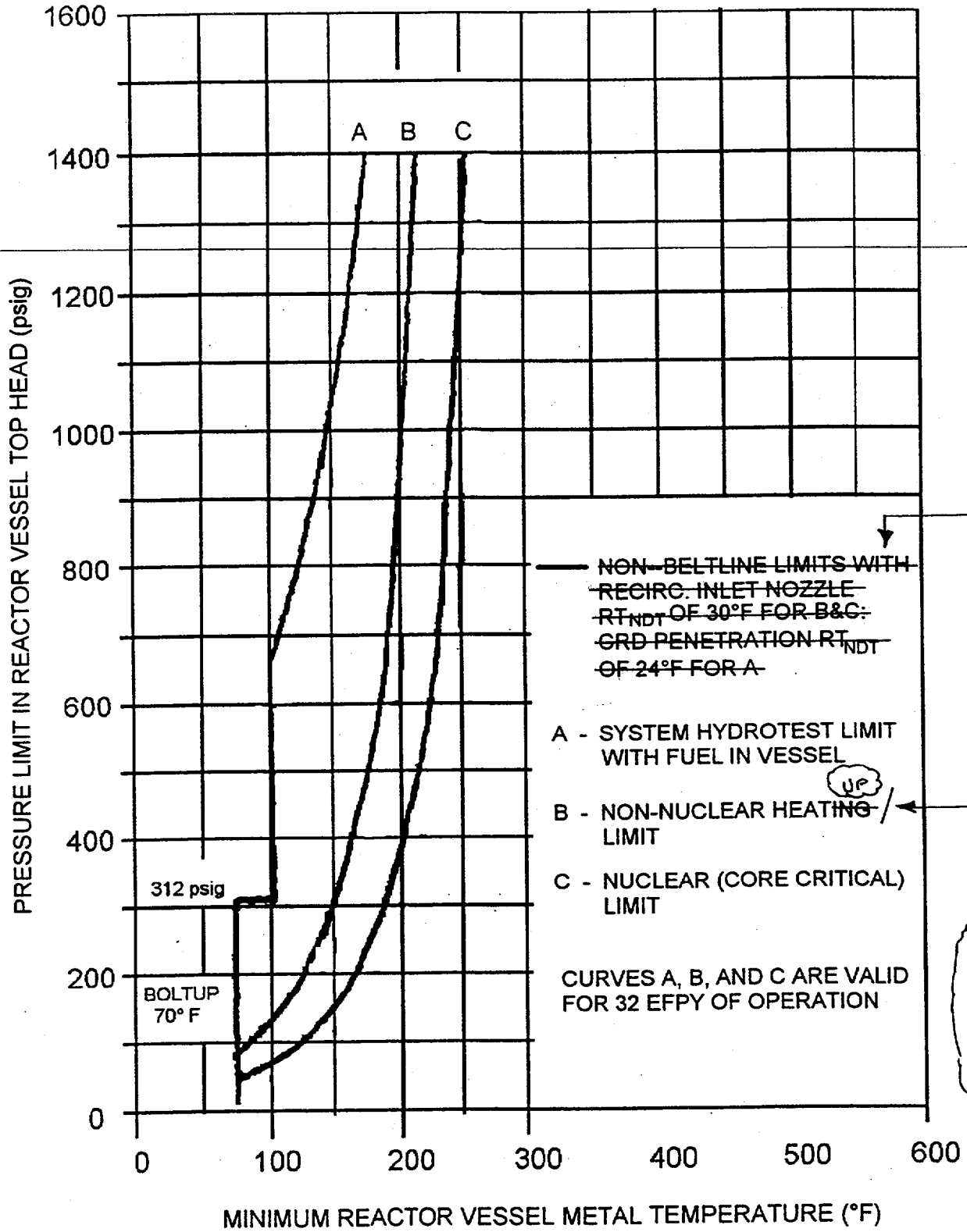
ATTACHMENT 3 TO PLA-5148

**MARKED-UP TECHNICAL SPECIFICATION
PAGES**



REACTOR VESSEL PRESSURE VS. MINIMUM VESSEL TEMPERATURE

Figure 3.4.10-1 (page 1 of 1)



A, B, C - Non-Beltline Limits
and Increased Appendix G
Requirements

COOLDOWN

REACTOR VESSEL PRESSURE VS. MINIMUM VESSEL TEMPERATURE

Figure 3.4.10-1 (page 1 of 1)

5.6 Reporting Requirements

and 2 (MARCH 1983), and Volume 1,
Supplement 3 (NOVEMBER 1990)

5.6.5 COLR (continued)

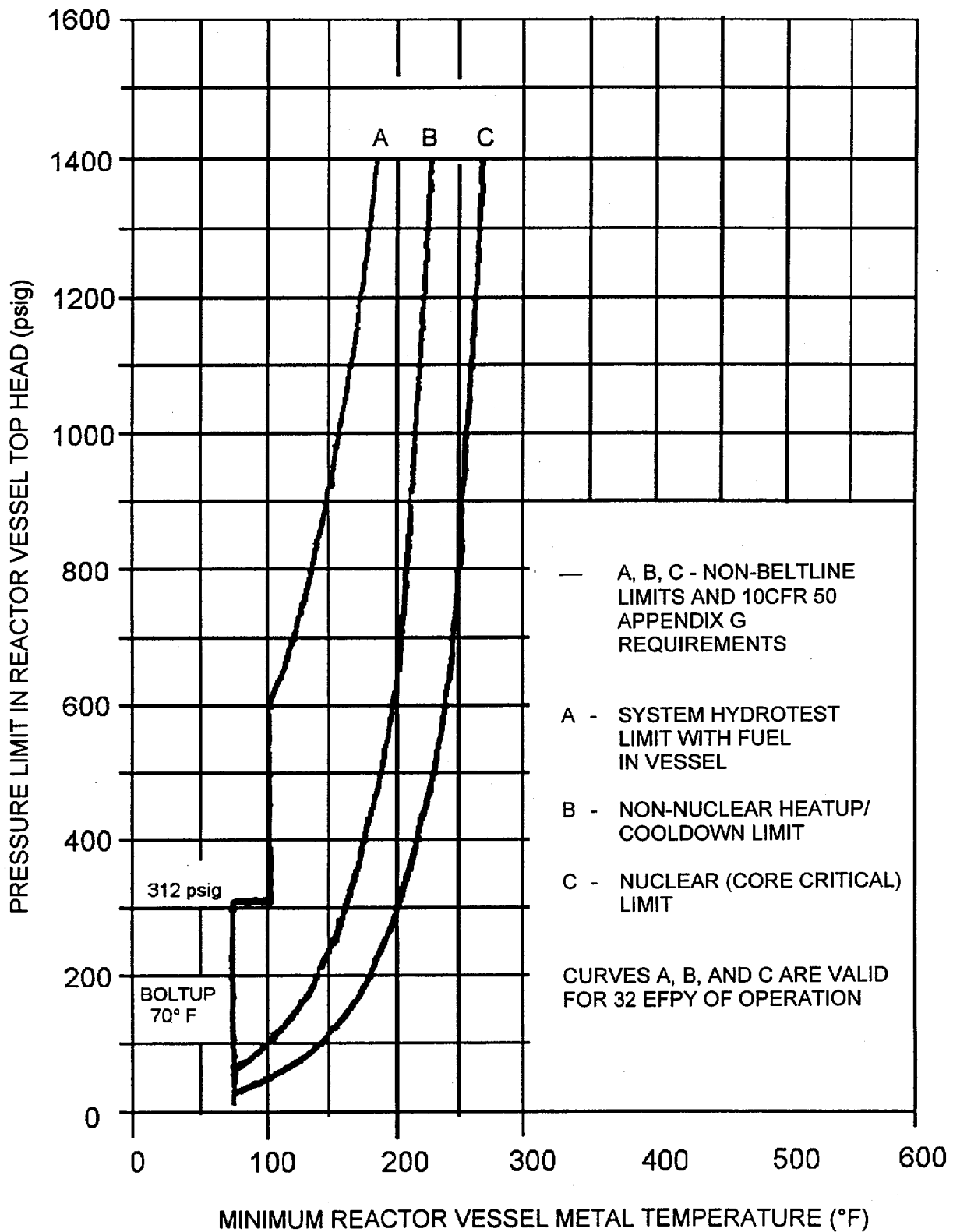
3. XN-NF-85-67(P)(A), Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel," Exxon Nuclear Company, Inc., September 1986.
4. XN-NF-80-19(A), Volume 1, and Volume 1 Supplements 1, 2, and 3, "Exxon Nuclear Methodology for Boiling Water Reactors: Neutronic Methods for Design and Analysis," Exxon Nuclear Company, Inc., March 1983.
5. ANF-524(P)(A), Revision 2 and Supplement 1, Revision 2, "Advanced Nuclear Fuels Corporation Critical Power Methodology for Boiling Water Reactors", November 1990.
6. ANF-1125(P)(A) and ANF-1125(P)(A), Supplement 1, "ANFB Critical Power Correlation", April 1990.
7. NEDC-32071P, "SAFER/GESTR-LOCA Loss of Coolant Accident Analysis," GE Nuclear Energy, May 1992.
8. NE-092-001A, Revision 1, "Licensing Topical Report for Power Uprate With Increased Core Flow," Pennsylvania Power & Light Company, December 1992.
9. NRC SER on PP&L Power Uprate LTR (November 30, 1993).
10. PL-NF-90-001, Supplement 1-A, "Application of Reactor Analysis Methods for BWR Design and Analysis: Loss of Feedwater Heating Changes and Use of RETRAN MOD 5.1," August 1995.
11. PL-NF-94-005-P-A, "Technical Basis for SPC 9x9-2 Extended Fuel Exposure at Susquehanna SES", January, 1995.
12. NEDE-24011-P-A-10, "General Electric Standard Application For Reactor Fuel, February, 1991.
13. PL-NF-90-001, Supplement 2-A, "Application of Reactor Analysis Methods for BWR Design and Analysis: CASMO-3G Code and ANFB Critical Power Correlation", July 1996.

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ATTACHMENT 4 TO PLA-5148

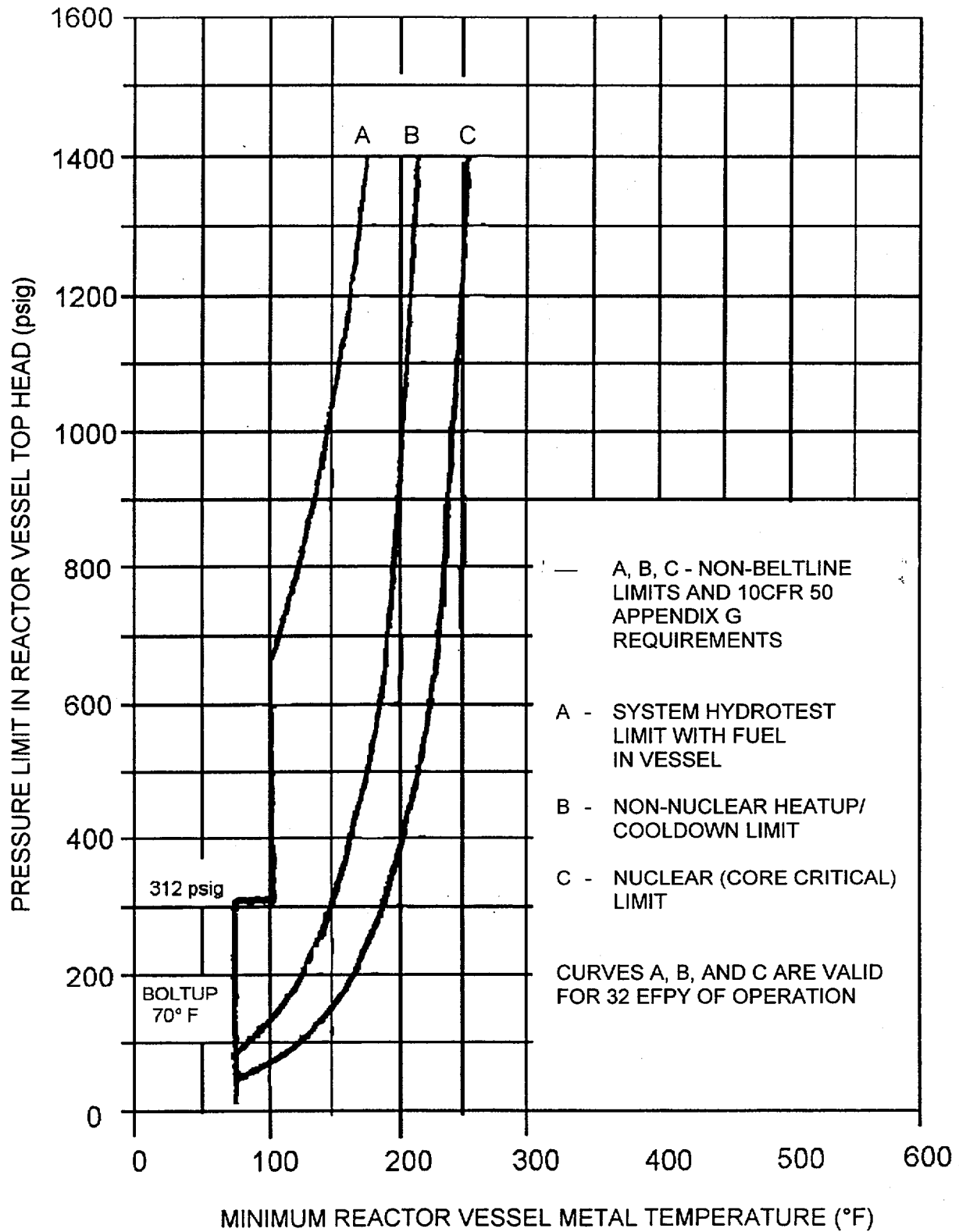
**“CAMERA-READY” TECHNICAL SPECIFICATION
PAGES**

**NOTE: “CAMERA-READY” PAGES ARE
BEING PROCESSED**



REACTOR VESSEL PRESSURE VS. MINIMUM VESSEL TEMPERATURE

Figure 3.4.10-1 (page 1 of 1)



REACTOR VESSEL PRESSURE VS. MINIMUM VESSEL TEMPERATURE

Figure 3.4.10-1 (page 1 of 1)

5.6 Reporting Requirements

5.6.5 COLR (continued)

3. XN-NF-85-67(P)(A), Revision 1, "Generic Mechanical Design for Exxon Nuclear Jet Pump BWR Reload Fuel, "Exxon Nuclear Company, Inc., September 1986.
4. XN-NF-80-19(A), Volume 1, and Volume 1 Supplements 1 and 2 (March 1983), and Volume 1, Supplement 3 (November 1990), "Exxon Nuclear Methodology for Boiling Water Reactors: Neutronic Methods for Design and Analysis," Exxon Nuclear Company, Inc.
5. ANF-524(P)(A), Revision 2 and Supplement 1, Revision 2, "Advanced Nuclear Fuels Corporation Critical Power Methodology for Boiling Water Reactors", November 1990.
6. ANF-1125(P)(A) and ANF-1125(P)(A), Supplement 1, "ANFB Critical Power Correlation", April 1990.
7. NEDC-32071P, "SAFER/GESTR-LOCA Loss of Coolant Accident Analysis," GE Nuclear Energy, May 1992.
8. NE-092-001A, Revision 1, "Licensing Topical Report for Power Uprate With Increased Core Flow," Pennsylvania Power & Light Company, December 1992.
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11. PL-NF-94-005-P-A, "Technical Basis for SPC 9x9-2 Extended Fuel Exposure at Susquehanna SES", January, 1995.
12. NEDE-24011-P-A-10, "General Electric Standard Application For Reactor Fuel, February, 1991.
13. PL-NF-90-001, Supplement 2-A, "Application of Reactor Analysis Methods for BWR Design and Analysis: CASMO-3G Code and ANFB Critical Power Correlation", July 1996.

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