



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

LR-N00-0003

JAN 17 2000

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 1
DOCKET NO. 50-272**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for December 1999 is attached.

Sincerely,

A handwritten signature in black ink that reads "M. B. Bezilla". The signature is fluid and cursive.

M. B. Bezilla
Vice President - Operations

/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

JE 24 1/2

The power is in your hands.

DOCKET NO.: 50-272
 UNIT: Salem 1
 DATE: 1/15/00
 COMPLETED BY: R. Knieriem
 TELEPHONE: (856) 339-1782

Reporting Period: December 1999

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)
 No. of hours reactor was critical
 No. of hours generator was on line (service hours)
 Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
744	7724	118545
744	7664	114251
0	0	0
821727	8009172	114621335

UNIT SHUTDOWNS

NO	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F Administrative
- G- Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 1 began the month of December 1999, operating at full power. Full power operation continued until December 31, when power was reduced to 80% as a contingency in anticipation of Y2K related problems affecting the power grid. Salem Unit 1 completed the month at 80% power.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 1 GENERATING STATION

MONTH: December 1999

The following items completed during **December 1999** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

Modification 1EC-3668, Pkg. 2, Rev. 0, Generic Letter 96-06 Modifications

This modification improved the performance of the Containment Fan Coil Unit (CFCU) Service Water Flow Control valves (SW223) through enhancements to the valve stroke time and the valve control logic.

Review of this modification under 10CFR50.59 was required because the modification to the SW 223 valves constituted a change to the facility as described in the SAR. This upgrade improved the operation of the SW223 valves and enhanced their ability to carry out their design function. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes

were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Modification 1EE-0042, Pkg. 1, Rev. 0, Containment Lighting Breaker Replacement

This modification replaced the 400 and 250 amp Penetration Protection breakers 1BKR2FL4YY and 2FL4Y with a design that would provide adequate protection to the downstream cabling and penetration feed-throughs in the event of a faulted condition less severe than the maximum credited fault.

Review of this modification under 10CFR50.59 was required because the replacement of the 400 and 250 amp Penetration Protection breakers constituted a change to the facility as described in the SAR. This upgrade enhanced the protection provided by these breakers and their ability to carry out their design function. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Modification 1EE-0390, Pkg. 6, Rev. 0, 16 Service Water Traveling Screen Replacement

This modification improved the existing Service Water Traveling Screens by replacing the screen material with a new Smooth-Tex design and the existing three nozzle type screen wash headers were replaced with new assemblies having six. The modification also added air-operated valves to isolate the safety-related portion of the Service Water system from the non-Safety-related screen wash portion of the system.

Review of this modification under 10CFR50.59 was required because the upgrade of the 16 Service Water Traveling Screen constituted a change to the facility as described in the SAR. This upgrade improved the operation of the traveling screens and enhanced their ability to carry out their design function. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Modification 7000441, Permanent Plugging of Floor Drains

This modification installed plates in floor drains in seven locations in Salem Unit 1. The plates were installed to plug the open drains in order to prevent steam flow from a potential Main Steam Line Break (MSLB) from communicating through the drains to mild areas in the Auxiliary Building.

Review of this modification under 10CFR50.59 was required because the installation of the plates in the floor drains constituted a change to the facility as described in the SAR. This change will assure the preservation of environmental conditions around equipment required for safety in the event of a MSLB. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Temporary Modifications - Summary of Safety Evaluations

There were no changes in this category implemented during December 1999.

Procedures - Summary of Safety Evaluations

Procedure NC.NA-AP.ZZ-0024(Q), Radiation Protection Program

This procedure revised the requirement for a termination whole body count for every radiologically controlled area worker to allow a more selective approach based on revised regulatory requirements and current technology.

Review of this procedure change under 10CFR50.59 was required because the revision of the requirement to perform a termination whole body count on every radiologically controlled area worker constituted a change to the facility as described in the SAR. This change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

UFSAR Change Notices - Summary of Safety Evaluations

There were no changes in this category implemented during December 1999.

Deficiency Reports - Summary of Safety Evaluations

There were no changes in this category implemented during December 1999.

Other - Summary of Safety Evaluations

There were no changes in this category implemented during December 1999.