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**Nuclear Business Unit**

JAN 17 2000

LR-N00019

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Gentlemen:

**MONTHLY OPERATING REPORT  
HOPE CREEK GENERATING STATION UNIT 1  
DOCKET NO. 50-354**

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for **December 1999** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is the summary of changes, tests, and experiments that were implemented during **December 1999**.

Sincerely,

A handwritten signature in black ink that reads "Mark B. Bezilla".

Mark B. Bezilla  
Vice President - Operations

RAR  
Attachments

C Distribution

*JED 4/1*

The power is in your hands.

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DOCKET NO.: 50-354  
 UNIT: Hope Creek  
 DATE: 1/14/00  
 COMPLETED BY: F. Todd  
 TELEPHONE: (856) 339-1316

Reporting Period December 1999

**OPERATING DATA REPORT**

**Design Electrical Rating (MWe-Net)**  
**Maximum Dependable Capacity (MWe-Net)**  
  
**No. of hours reactor was critical**  
**No. of hours generator was on line (service hours)**  
**Unit reserve shutdown hours**  
**Net Electrical Energy (MWH)**

<b>1067</b>		
<b>1031</b>		
<b>Month</b>	<b>Year-to-date</b>	<b>Cumulative</b>
<b>744</b>	<b>7598</b>	<b>96727</b>
<b>744</b>	<b>7538</b>	<b>95147</b>
<b>0</b>	<b>0</b>	<b>0</b>
<b>761458</b>	<b>7701078</b>	<b>96298599</b>

**UNIT SHUTDOWNS**

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

DOCKET NO.: 50-354  
UNIT: Hope Creek  
DATE: 1/14/00  
COMPLETED BY: R. Ritzman  
TELEPHONE: (856) 339-1445

### **Summary Of Monthly Operating Experience**

- Hope Creek entered the month of December at approximately 100% reactor power.
- Power was reduced on December 10 for hydraulic control unit maintenance and was restored to approximately 100% on December 13.
- On December 31, power was reduced as a Y2K roll-over contingency.
- At the end of the month, Hope Creek completed 121 days of continuous power operation.

DOCKET NO.: 50-354  
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TELEPHONE: (856) 339-1445

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE HOPE CREEK GENERATING STATION**

**MONTH December 1999**

The following items completed during **December 1999** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

**Design Changes    Summary of Safety Evaluations**

There were no reportable changes in this category implemented during December 1999.

**Temporary Modifications    Summary of Safety Evaluations**

There were no reportable changes in this category implemented during December 1999.

09/20/00

**SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS**  
**FOR THE HOPE CREEK GENERATING STATION – Cont'd**

**Procedures Summary of Safety Evaluations**

**HC.OP-SO.BF-0001(Q), Control Rod Drive Hydraulic System Operation, Revision 17.** This procedural revision included the ability to operate flow control valves 1BFFV-F002A and B in the manual mode versus the automatic mode as described in the UFSAR. The new alignment is to be used only during corrective maintenance and calibration activities. This alignment allows plant technicians to locally adjust and calibrate loop instrumentation.

This procedure revision does not impact the ability of the Control Rod Drive system to perform its design function. It does not impact the ability of the plant to mitigate any operational transients or postulated design basis accidents. This procedure revision does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety. Therefore it does not involve an Unreviewed Safety Question.

**UFSAR Change Notices Summary of Safety Evaluations**

**UFSAR Change Notice HCN 99-071, Use of Helium and Sulfur Hexafluoride to Locate Condenser Circulating Water Leaks.** This UFSAR change notice allows for the installation of taps on the condenser inlet piping. The taps will be used for tracer gas injection into the circulating water system. The tracer gas will be used to identify and define the location of condenser leaks.

This proposal does not impact the ability of the plant to meet its design basis or to mitigate any operational transients or postulated design basis accidents. This procedure revision does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety. Therefore it does not involve an Unreviewed Safety Question.

**UFSAR Change Notice HCN 99-072, Shift Responsibility for Contractor Oversight from Maintenance to Planning.** This UFSAR change notice shifts the responsibility for oversight of contract maintenance services from the Maintenance Department to Planning in the Plant Support Department. This change notice is administrative in nature and does not affect the method of contractor oversight.

The implementation of this change notice does not impact any plant equipment or processes. The appropriate contractor oversight functions will continue to be performed. This change notice does not impact any operational transients or postulated design basis accidents. This change notice does not increase the possibility or consequences of any accident or malfunction, does not reduce the margin of safety, and; therefore, does not involve an Unreviewed Safety Question.