



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D.C. 20555-0001

January 12, 2000

Mr. L. W. Myers  
Senior Vice President  
Beaver Valley Power Station  
Post Office Box 4  
Shippingport, PA 15077

SUBJECT: BEAVER VALLEY 1 AND 2 - APPROVAL OF PROPOSED REVISIONS TO THE  
QUALITY ASSURANCE PROGRAM DESCRIPTION (TAC NOS. MA4992 AND  
MA4993)

Dear Mr. Myers:

By letter dated March 6, 1999 (L-99-015), as supplemented by letters dated June 25, 1999 (L-99-093), and November 19, 1999 (L-99-174), the Duquesne Light Company (DLC) submitted a request for approval of a proposed change in the Beaver Valley Power Station, Unit Nos. 1 and 2 (BVPS-1 and BVPS-2) Quality Assurance (QA) Program Description in Chapter 17.2 of the BVPS-2 Updated Final Safety Analysis Report (UFSAR). The BVPS-1 and BVPS-2 QA Program description is applicable to both units and is contained in the BVPS-2 UFSAR. Specifically, the proposed change would limit the required Onsite Safety Committee (OSC) reviews of procedures to those requiring a safety evaluation in accordance with 10 CFR Part 50, Section 59 (10 CFR 50.59). Implicit to this change would be the substitution of procedure reviews by independent qualified reviewers and approval by responsible discipline Managers, or their designees, in place of the OSC review when a completed 10 CFR 50.59 safety evaluation is not required.

On the dates of the March 6, June 25, and November 19, 1999, letters, DLC was the licensed operator for BVPS-1 and BVPS-2. On December 3, 1999, DLC's ownership interests in both BVPS-1 and BVPS-2 were transferred to the Pennsylvania Power Company, and DLC's operating authority for BVPS-1 and BVPS-2 was transferred to FirstEnergy Nuclear Operating Company (FENOC). By letter dated December 13, 1999, FENOC requested that the Nuclear Regulatory Commission (NRC) continue to review and act upon all requests before the Commission which had been submitted by DLC.

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Accordingly, the NRC staff has completed its review of this proposed change to the QA Program Description and has concluded that the proposed change will continue to satisfy the guidance in NUREG-0800, "Standard Review Plan," Section 17.2, and the requirements of 10 CFR Part 50, Appendix B, and is, therefore, acceptable. The enclosed safety evaluation documents the basis for our conclusion that the proposed change to the QA Program Description is acceptable.

Please contact me at (301) 415-1427 if you have any questions regarding this issue.

Sincerely,

/RA/

Daniel S. Collins, Project Manager, Section 1  
Project Directorate I  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket Nos. 50-334 and 50-412

Enclosure: Safety Evaluation

cc w/encl: See next page

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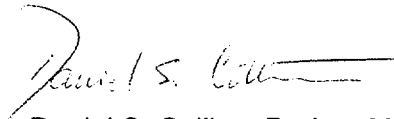
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Daniel S. Collins, Project Manager, Section 1  
Project Directorate I  
Division of Licensing Project Management  
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cc w/encl: See next page

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