



***** SAFEGUARDS INFORMATION *****
UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 28, 1999

Mr. Robert G. Byram
Senior Vice President - Generation
and Chief Nuclear Officer
PP&L, Inc.
2 North Ninth Street
Allentown, Pennsylvania 18101

SUBJECT: NRC INSPECTION 50-387/99201 AND 50-388/99201 (OPERATIONAL SAFEGUARDS RESPONSE EVALUATION)

Dear Mr. Byram:

During August 2 through 5, 1999, NRC's Office of Nuclear Reactor Regulation performed an Operational Safeguards Response Evaluation (OSRE) at the Susquehanna Steam Electric Station, Units 1 and 2. A final exit meeting was held by telephone conference on October 13, 1999, to summarize the findings of the OSRE. The enclosed report presents the scope and results of that inspection.

The primary purpose of the OSRE was to assess your ability to respond to an external threat. The OSRE also assessed the interaction between operations and security in establishing priorities for protection of equipment and the protective strategies used. The OSRE included a safety/safeguards interface review to ensure that safeguards measures did not adversely impact safe operations of the facility.

The licensee's performance during this evaluation indicated an excellent overall contingency response capability. The OSRE team noted significant operations involvement in the planning and exercising of the contingency response capability, and concluded that the protective strategy for Susquehanna was being effectively implemented and that the response force demonstrated excellent capabilities in protecting public health and safety against the NRC design basis threat (DBT). The exercises also demonstrated that the security response force could provide a protected, safe environment for the operators to operate and control the reactor within their normal training and qualifications. The team also concluded that effective provisions were in place to ensure that safeguards measures did not adversely affect the safe operations of the facility.

The enclosed report does not convey any new regulatory requirement. The enclosure, which contains safeguards information of a type specified in 10 CFR 73.21, will not be

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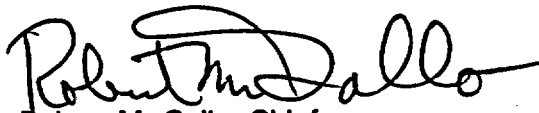
Mr. R. Byram

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No response to this letter is required. Thank you for your cooperation.

Sincerely,



Robert M. Gallo, Chief
Operator Licensing, Human Performance and Plant
Support Branch
Division of Inspection Program Management
Office of Nuclear Reactor Regulation

Docket Nos: 50-387
50-388

License Nos: NPF-14
NPF-22

Enclosure: NRC Inspection Report No. 50-387/99201 and 50-388/99201 (Operational Safeguards Response Evaluation)

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Mr. R. Byram

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No response to this letter is required. ^{PUBLIC DOCUMENT ROOM} Thank you for your cooperation.

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Sincerely,

Original signed by:
Robert M. Gallo, Chief
Operator Licensing, Human Performance and Plant
Support Branch
Division of Inspection Program Management
Office of Nuclear Reactor Regulation

Docket Nos: 50-387 and 50-388
License Nos: NPF-14 and NPF-22
Enclosure: NRC Inspection Report Nos. 50-387/99201 and 50-388/99201 (Operational Safeguards Response Evaluation)

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