

January 13, 2000

MEMORANDUM TO: Stuart A. Richards, Director
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Jack Cushing, Project Manager, Section 2 **/RA/**
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: FORTHCOMING MEETING WITH COMBUSTION ENGINEERING
OWNERS GROUP

DATE & TIME: Thursday, January 27, 2000
1:00 p.m. - 4:00 p.m.

LOCATION: U. S. Nuclear Regulatory Commission
One White Flint North
11555 Rockville Pike, Room O-13B2
Rockville, Maryland

PURPOSE: To discuss extending the reactor vessel inservice inspection beyond the
current 10 year requirement. For more information see web site:
<[A HREF="PMNS/NRR5.html"](PMNS/NRR5.html)>web.

PARTICIPANTS:* NRC CEOG
J. Strosnider J. Ghergurovich
B. Bateman P. Riccardella
K. Wichman C. Brinkman
A. Hiser C. Hoffman
M. Reinhart D. Aryes
B. Elliot J. Lareau
M. Mayfield E. Scherer
B. Herman S. Shaw, et. al.
J. Cushing, et. al.

Project No. 692

Attachment: Draft Agenda

cc w/att: See next page

*Meeting between the NRC technical staff and applicants or licensees are opened for interested members of the public, petitioners, intervenors, or other parties to attend as observers pursuant to "Commission Policy Statement on Staff Meetings Open to the Public" 59 Federal Register 48340, 9/20/94. However, the portions of the meeting dealing with the VIPER code will be closed to protect the CE Owners 's Group proprietary information. Members of the public who wish to attend the open portion of the meeting should contact Jack Cushing at (301) 415-1424.

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GROUP AND NRC ON JANUARY 27, 2000

Dated: January 13, 2000

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AGENDA FOR EXTENDING THE REACTOR VESSEL ISI INTERVAL

Introduction

General discussion of the purpose of the meeting after which the meeting will be closed to discuss proprietary information.

Description of Methodology

Material Properties - developing appropriate subregions of the beltline and defining material properties for each subregion.

Transient Analysis - Definition of the set transients and discussion of the use of existing analyses and extrapolations and adjustments of those analyses.

Probabilistic Risk Assessment - Discussion of probabilistic safety assessment aspects.

Flaw Distribution - Determination of beltline flaw distribution and uncertainty as input to VIPER code.

Fracture Mechanics - Develop and formulate physical models for fatigue crack growth.

VIPER Code Issues - Discuss modifications and use of this code to calculate reactor vessel failure probabilities vs. inservice inspection intervals.