



Omaha Public Power District
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Omaha, Nebraska 68102-2247

January 4, 2000
LIC-99-0129

U. S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

- References:
1. Docket No. 50-285
 2. Letter from OPPD (W. G. Gates) to NRC (Document Control Desk) dated June 23, 1993 (LIC-93-0119)
 3. Letter from OPPD (W. G. Gates) to NRC (Document Control Desk) dated January 13, 1994 (LIC-93-0304)
 4. Letter from OPPD (T. L. Patterson) to NRC (Document Control Desk) dated June 12, 1995 (LIC-95-0123)
 5. Letter from OPPD (S. K. Gambhir) to NRC (Document Control Desk) dated January 10, 1997 (LIC-97-0001)
 6. Letter from OPPD (S. K. Gambhir) to NRC (Document Control Desk) dated January 30, 1998 (LIC-98-0009)
 7. Letter from OPPD (S. K. Gambhir) to NRC (Document Control Desk) dated July 31, 1998 (LIC-98-0097)

SUBJECT: Average Load Capacity Factor for Fort Calhoun Station (FCS) Cycle 18

The Omaha Public Power District (OPPD) monitors the long term load capacity factor for FCS on a cycle-by-cycle basis to ensure that the current fluence/pressurized thermal shock (PTS) and other fluence dependent analyses remain valid. This is required pursuant to Condition 3.D of FCS Operating License DPR-40. In Reference 3, OPPD committed to notify the NRC of the average load capacity factor achieved since the start of Cycle 15. References 4, 5 and 7 provided the data for Cycles 15, 16 and 17, respectively. The load capacity factor for Cycle 18 operation (including the refueling outage) was 0.90.

A revised fluence evaluation utilizing the ENDF-B/VI cross sections (per Reference 2) was completed in 1995. This evaluation, in conjunction with the load capacity factors of completed fuel cycles and projected capacity factors of future cycles, continues to be utilized to assess the projected end of life 10 CFR 50.61 RT_{PTS} value. These projections assume future fuel cycle load capacity factors of 0.85. This information was detailed in the Reference 6 Application for Amendment of Operating License, which proposed to delete the license condition described above. OPPD has subsequently supplemented this application and continues to discuss the issue with the NRC Staff.

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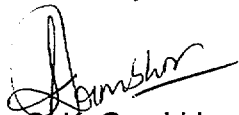
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The current pressure-temperature (P-T) limit curves/limits in the FCS Technical Specifications are for 20 Effective Full Power Years. These curves/limits are based on conservative fluence and chemistry values and analyses, and were approved by the NRC in Amendment 161 to FCS Facility Operating License No. DPR-40. OPPD has reviewed the P-T curves/limits and the reactor vessel beltline materials based on the 1995 revised fluence evaluation and actual operation through the end of Cycle 18, and has determined that the current P-T curves/limits remain valid for Cycle 19 operation.

OPPD plans future discussions and correspondence with the NRC Staff on fluence and PTS issues. OPPD intends to demonstrate that the welds and plate material in the reactor vessel beltline region are projected to remain below the 10 CFR 50.61 PTS screening criteria well beyond the end of the current operating license term.

Please contact me if you have any questions.

Sincerely,



S. K. Gambhir
Division Manager
Nuclear Operations

TCM/tcm

Attachment

c: E. W. Merschoff, NRC Regional Administrator, Region IV
L. R. Wharton, NRC Project Manager
W. C. Walker, NRC Senior Resident Inspector
Winston & Strawn