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Nuclear Business Unit

DEC 27 1999

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United States Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SPECIAL REPORT 99-04 HOPE CREEK GENERATING STATION FACILITY OPERATING LICENSE NPF-57 DOCKET NO. 50-354

Gentlemen:

This Special Report is being submitted pursuant to the requirements of Hope Creek Technical Specification 3.3.7.5-1, Action 81 b, due to the South Plant Vent (SPV) High Range Noble Gas Monitor being inoperable for more than 72 hours.

On December 18, 1999, a loss of isokinetic flow was noted for the South Plant Vent (SPV). This flow signal provides an input to the SPV radiation monitoring system.

Technical Specification Action Statements 3.3.7.5, "Accident Monitoring Instrumentation", and 3.3.7.11, "Radioactive Gaseous Effluent Monitoring Instrumentation", were entered, and compensatory measures were initiated in accordance with plant Technical Specifications.

The SPV radiation monitoring system (RMS) inoperability was caused by an out of calibration flow transmitter and a failed vacuum pump diaphragm. The pump diaphragm was replaced, the flow transmitter was recalibrated and the SPV RMS was retested satisfactorily, and the system was restored to OPERABLE status on December 22, 1999.

Should you have any questions or comments on this transmittal, please contact John Nagle at 856-339-3171.

1. Bezilla

Vice President - Operations

TEDD



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December 31, 1999

PG&E Letter DCL-99-169

U. S. Nuclear Regulatory Commission ATTN: Document Control Desk Washington, DC 20555-0001

Docket No. 50-275, OL-DPR-80
Docket No. 50-323, OL-DPR-82
Diablo Canyon Units 1 and 2
Withdrawal of License Amendment Request 99-01, "I

Withdrawal of License Amendment Request 99-01, "Unreviewed Safety Question - Spurious Operation of the Safety Injection System at Power"

Dear Commissioners and Staff:

The purpose of this letter is to withdraw License Amendment Request (LAR) 99-01, "Unreviewed Safety Question - Spurious Operation of the Safety Injection System at Power," submitted by PG&E Letter DCL-99-071, dated May 21, 1999. LAR 99-01 requested NRC review and approval of a revised analysis for the spurious operation of the safety injection (SI) system at power event crediting automatic actuation of the Class 1 power operated relief valves (PORVs), instead of the pressurizer safety valves (PSVs), to limit reactor coolant system pressure. No credit is taken in the current Final Safety Analysis Report Update for automatic operation of the PORVs, which function to limit undesirable opening of the PSVs, since part of the automatic actuation circuitry is Instrument Class II.

After discussions with the NRC staff regarding the proposed LAR, PG&E has decided to pursue other options to resolve the issues relating to the spurious operation of the SI system at power event.

Sincerely,

Lawrence F. Womack

Document Control Desk December 31, 1999 Page 2

cc: Edgar Bailey, DHS
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Diablo Distribution