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NUCLEAR REGULATORY COMMISSION
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December 29, 1999

MEMORANDUM TO: Stuart A. Richards, Director
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

FROM: Leonard N. Olshan, Project Manager
Project Directorate II-1
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

SUBJECT: SUMMARY OF MEETING WITH EPRI TO DISCUSS FUTURE USE OF RISK-INFORMED TECHNOLOGY

On December 9, 1999, the Nuclear Regulatory Commission (NRC) and the Electric Power Research Institute (EPRI) met in Rockville, Maryland to discuss EPRI's plans for future use of risk-informed technology. Attachment 1 is a list of the meeting attendees. The slides presented by EPRI are available under ADAMS accession number ML99347027.

EPRI is estimating that, in the year 2000, approximately twenty risk-informed inservice inspection (RI-ISI) submittals will be sent to the NRC. The staff stated that, based on its inquiries in September 1999, it was not expecting such a large number of submittals. EPRI responded that the large increase in licensee interest is due to the issuance in October 1999 of the staff's safety evaluation on the EPRI RI-ISI topical report. The staff indicated that it would be very difficult to accommodate this many reviews within one year. The staff requested that EPRI consider bringing such resource-intensive reviews to the attention of the staff in a more timely way for resource planning.

EPRI proposed to apply RI-ISI to high energy piping in the break exclusion region as defined by Standard Review Plan (SRP) 3.6.2. The staff noted that, for plants licensed after 1975 (e.g., Nine Mile Point 2), eliminating postulated breaks using RI-ISI might require an exemption from General Design Criterion 4. Furthermore, the manner in which indirect spatial effects (e.g., pipe whip and jet impingement) have been considered for RI-ISI might not provide a sufficient basis for eliminating examination of the welds in the break exclusion region because the RI-ISI approach for considering indirect spatial effects is not as comprehensive as the pipe-break criteria recommended in SRP 3.6.2.

EPRI discussed its plans to develop performance-based criteria for containment ISI. The staff noted that because containment ISI (IWE/IWL) requirements are relatively new, and licensees are inspecting their containments for the first time in accordance with 10 CFR 50.55a requirements, it would be useful for the staff to understand first-hand the types of degradation licensees are finding in operating plants as well as the difficulties licensees are experiencing in performing certain inspections. This understanding will enable the staff to develop, with the industry, performance-based criteria for containment ISI. EPRI agreed to pursue this issue further and explore the possibilty of NRC staff accompanying licensees on selected containment inspection activities.

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PDR REVGP EPRI

EPRI also discussed RI-ISI as it may relate to Section VIII, Appendix VIII on performance demonstration of ultrasonic examination techniques, Generic Letter 88-01 on intergrannular stress corrosion cracking in BWR piping, the Materials Reliability Project activities on thermal fatigue of small bore piping, and updating of the EPRI topical report.

Project No. 669

Attachment: Attendance List

cc w/att: See next page

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cc w/att: See next page

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MEETING TO DISCUSS EPRI'S FUTURE USE OF RISK-INFORMED TECHNOLOGY

DECEMBER 9, 1999

MEETING ATTENDEES

NRC

S. Ali
M. Cheok
S. Dembek
S. Dinsmore
D. Fischer
R. Hermann
S. Hou
R. Li
L. Olshan
S. Pullani
A. Serkiz
E. Sullivan
D. Terao

EPRI

F. Ammirato
J. Mitman
P. O'Regan
H. Stephens

OTHER

R. Fougarousse, Inservice Engineering
S. Kulat, Inservice Engineering
E. McClain, Niagara Mohawk
K. Hall, Entergy
S. Welp, Baltimore Gas & Electric
V. Dinitrijevic, Duke Engineering & Services
J. Hutchinson, Commonwealth Edison
S. Chesworth, Structural Integrity
D. Raleigh, Bechtel
R. Huston, Licensing Support Services
H. Fish, New York Power Authority

Attachment