

Westinghouse Electric Company Nuclear Power Plants P.O. Box 355 Pittsburgh, Pennsylvania 15230-0355 USA

U.S. Nuclear Regulatory Commission ATTENTION: Document Control Desk Washington, D.C. 20555 Direct tel: 412-374-6206 Direct fax: 724-940-8505 e-mail: sisk1rb@westinghouse.com

Your ref: Docket No. 52-006 Our ref: DCP\_NRC\_002943

June 30, 2010

Subject: 10 CFR 50.46 Thirty(30) Day Report for the AP1000 Standard Plant Design

Pursuant to 10 CFR 50.46, "Acceptance criteria for emergency core cooling systems for light-water reactors," Westinghouse Electric Company is submitting this report to document any emergency core cooling system (ECCS) evaluation model changes or errors that affect the temperature calculation for the AP1000 Standard Plant Design. A 30 day report is required if the PCT error assessed is greater than 50°F.

This letter is to transmit 10 CFR 50.46 reporting information for errors discovered in the AP1000 Design Control Document (DCD) small-break loss of coolant accident (LOCA) analysis-of-record (AOR) for the Loss of Coolant Accident (LOCA) 10-inch Transient Adiabatic Heat-up Calculation. While performing analyses to evaluate the AP1000 first core, errors were discovered in the Small-Break LOCA (SBLOCA) AOR. Westinghouse documented this in the Westinghouse Corrective Action Process system (CAPs) to track resolution of the errors. Calculations documenting the estimate of effect for these errors have been completed. 10 CFR 50.46 reporting information is attached with the revised rack-ups.

The 10-Inch Transient Adiabatic Heat up resulted in a PCT increase of 264°F for the Small Break Lossof-Coolant Accident (SBLOCA). The peak cladding temperature (PCT) is 1634°F for the SBLOCA and does not exceed the 10 CFR 50.46 (b)(l) acceptance criterion of 2200°F. The summary of the updated PCT margin allocations and their bases are provided in Enclosure 1. Westinghouse submitted the initial 10 CFR 50.46 report for the AP1000 Standard Plant Design in letter DCP/NRC2074, dated February 15, 2008. The limiting peak clad temperature of 2158°F for the BELOCA has not changed since the issuance of DCP/NRC2074.

The information included in this letter is generic and is expected to apply to all COL applications referencing the AP1000 Design Certification and Design Certification Amendment Application. By copy of this letter, COL Applicants are hereby notified of any changes or errors in the AP1000 Standard Design PCT as required by 10 CFR 50.46(a)(3)(iii).

DCP\_NRC\_002943 June 30, 2010 Page 2 of 2

Questions or requests for additional information related to content and preparation of this information should be directed to Westinghouse. Please send copies of such questions or requests to the prospective applicants for combined licenses referencing the AP1000 Design Certification. A representative for each applicant is included on the cc: list of letter.

Very truly yours,

for John Deblasic

Robert Sisk, Manager Licensing and Customer Interface Regulatory Affairs and Strategy

#### /Enclosure

cc:

1. 10 CFR 50.46 Thirty (30)Report

D. Jaffe	-	U.S. NRC		lΕ
E. McKenna	-	U.S. NRC		ΙE
S. K. Mitra	-	U.S. NRC		ΙE
T. Spink		TVA	-	ΙE
P. Hastings	-	Duke Power		E
R. Kitchen	-	Progress Energy		İΕ
A. Monroe	-	SCANA		ΙE
P. Jacobs	-	Florida' Power & Light	-	ΙE
C. Pierce	-	Southern Company	-	ΙE
E. Schmiech	-	Westinghouse	•	ΙE
G. Zinke	-	NuStart/Entergy		ΙE
R. Grumbir	-	NuStart		ΙE
D. Behnke	-	Westinghouse		ΙE
J. Monahan	-	Westinghouse		ΙE

## ENCLOSURE 1

## 10 CFR 50.46 Report

## Attachment

# AP1000 SBLOCA 10-INCH TRANSIENT ADIABATIC HEAT-UP CALCULATION (Non-Discretionary Change)

#### Background

Incorrect inputs were found to have been utilized in the calculation as the adiabatic rod heat-up during the 10-inch break analysis of the AP1000. The first error was related to inconsistent units being utilized in the equation for the rod average power. The second error was the usage of an incorrect cross-sectional area term to represent the cladding in the adiabatic heat-up calculation. These changes represent Non-Discretionary changes in accordance with Section 4.1.2 of WCAP-13451.

#### Affected Evaluation Model(s)

.

1985 Westinghouse Advanced Plant Small Break LOCA Evaluation Model with NOTRUMP

#### **Estimated Effect**

The estimated effect of these changes on the Adiabatic Rod heat-up for the 10-inch break is an increase in Peak Cladding Temperature of 264 °F.

### Westinghouse LOCA Peak Clad Temperature Summary for Appendix K Small Break

Plant Name: Utility Name: Revision Date:		AP1000 Westinghouse Nuclear Power Plants 6/29/2010									
Analysi	s Informati	<u>on</u>									
EM:	NOTR	UMP-AP	Analysis Date:	8/23/2002	Limiting Break Size:	10 Inch					
FQ:	2.6		FdH:	1.65							
Fuel:	RFA		SGTP (%):	10							
Notes:											
					Clad Temp (°	F) Ref.	Notes				
LICEN	SING BA	SIS			• •	,					
	Analysis-	Of-Record P	СТ	137	0 1	(a)					
РСТ А	SSESSME	NTS (Delta	PCT)								
	A. PRIOR	ECCS MO	DEL ASSESSMEN		0						
]	<b>B. PLANN</b> 1 . 1	ED PLANT	S	0							
	<b>C. 2010 E</b> (	CCS MODE	L ASSESSMENTS	26	4 2	(a)					
	<b>D. OTHEI</b> 11	<b>R*</b> None				0					
]	LICENSIN	NG BASIS P	PCT + PCT ASSES	SMENTS	<b>PCT =</b> 163	34					

\* It is recommended that the licensee determine if these PCT allocations should be considered with respect to

10 CFR 50.46 reporting requirements.

#### **Reference:**

1 . APP-GW-GL-700, Tier 2, Chapter 15, Rev. 3, "Design Certification Document: Accident Analysis" March 2003.

#### Notes:

(a) This is an adiabatic heat-up calculated PCT.